

NRC Form 204 (10-82)										U.S. NUCLEAR REGULATORY COMMISSION APPROVED ONS NO. 3180-0104 EXPIRES - 6/31/85									
LICENSEE EVENT REPORT (LER)																			
FACILITY NAME (1) Peach Bottom Atomic Power Station - Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 1 2 1 7 1 8					PAGE (3) 1 OF 0 1 3				
TITLE (4) Isolated Fuel Bundle in the Unit 3 Reactor Core																			
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)										
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)							
0 8	0 7	8 5	8 5	0 1 2		0 0	0 9	0 6				0 5 0 0 0 1 1							
												0 5 0 0 0 1 1							
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																	
0		20.402(a)			20.406(a)			60.73(a)(2)(iv)			73.71(a)								
POWER LEVEL (10)		20.408(a)(1)(i)			60.36(a)(1)			60.73(a)(2)(v)			73.71(a)								
0 0 0		20.408(a)(1)(ii)			60.36(a)(2)			60.73(a)(2)(vi)			OTHER (Specify in Appendix 90-10 and in Test, NRC Form 364A)								
		20.408(a)(1)(iii)			X 60.73(a)(2)(ii)			60.73(a)(2)(vii)(A)											
		20.408(a)(1)(iv)			60.73(a)(2)(iii)			60.73(a)(2)(viii)(B)											
		20.408(a)(1)(v)			60.73(a)(2)(iv)			60.73(a)(2)(ix)											
		20.408(a)(1)(vi)			60.73(a)(2)(v)			60.73(a)(2)(x)											
LICENSEE CONTACT FOR THIS LER (12)																			
NAME W. C. Birely, Senior Engineer - Licensing Section										TELEPHONE NUMBER 2 1 5 8 4 1 7 5 0 4 1 8									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC									
				N															
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR					
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO									
ABSTRACT (Summarize in 1000 words, i.e., approximately fifteen single-space typewritten lines) (16)																			
<p><u>Abstract:</u> 3-85-12</p> <p>On August 7, 1985 at 8:30 a.m., while the Unit 3 reactor fuel core was being off-loaded, a peripheral fuel bundle was isolated from a Source Range Monitor (SRM). The fuel handling sequence being used to off-load the core had been temporarily changed to allow continued fuel moves because of an inoperable SRM in a quadrant of the core. The temporary procedure change of the sequence did not require, as it should have, the peripheral fuel bundle to be the first bundle removed from that quadrant. When this condition was discovered, another change was implemented to allow the peripheral fuel bundle to be moved to the fuel pool. After this fuel bundle was removed, fuel moves were suspended until the cause of the event had been evaluated. Although the SRM might not have been able to monitor the isolated fuel, safety concerns are minimal because the core had undergone extended core life and the individual fuel bundle had been in the core for three fuel cycles.</p>																			

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMS NO. 3190-0104

EXPIRES 8/31/86

FACILITY NAME (1) Peach Bottom Atomic Power Station - Unit 3	DOCKET NUMBER (2) 05000278815	LER NUMBER (3)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		01	12	01	02	OF	03

TEXT (If more space is required, use additional NRC Form 366a (17))

Description of the Event:

On August 7, 1985 with the Unit 3 reactor fuel being off-loaded for refueling and maintenance, a fuel bundle was isolated from an SRM. This appears to violate Technical Specification 3.10.B.5, which states: "During the unloading and reloading of fuel, intermediate arrays of fuel shall always contain at least one SRM", because the SRM may not have been able to monitor that fuel bundle. The isolated fuel bundle (core location 15-04) was to have been the first fuel bundle removed from the quadrant; however, a temporary procedure change to the fuel handling sequence (FH-6C Appendix A, "Core Component Transfer Authorization Sheet") bypassed this bundle.

This condition was discovered immediately after the last adjacent fuel bundle (core location 17-04) had been removed from the core. Upon discovery, a change was implemented to allow the isolated bundle to be moved to the fuel pool.

Cause of the Event:

This event was caused by personnel error. A block of moves were relocated in the fuel handling sequence as a result of a temporary change. This block of moves was offset by one bundle from the correct block resulting in the peripheral fuel bundle not being off-loaded instead of being the first bundle off-loaded in that quadrant.

Consequences of the Event:

The consequences of this unmonitored fuel bundle are minimal. All control rods were full-in; the shutdown margin for the reactor is such that any single rod drop cannot bring the reactor to criticality; the bundle had been in the core for three cycles and is depleted; and the bundle was unmonitored for only three hours.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (5)

PAGE (3)

Peach Bottom Atomic Power
Station - Unit 3

YEAR

SEQUENTIAL
NUMBERREVISION
NUMBER

0 5 0 0 0 2 7 8 8 5 - 0 1 2 - 0 0 0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 365A (17))

Corrective Actions:

The fuel handling procedure was temporarily changed to permit the isolated fuel bundle to be moved to the fuel pool.

Actions Taken To Prevent Recurrence

Temporary changes of fuel handling sequences have been suspended until appropriate temporary procedure change criteria have been defined and approved for such sequences. The entire fuel move sheet was verified to ensure that it was complete and accurate.

Previous Similar Occurrences:

None.

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September 6, 1985

Docket No. 50-278

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Washington, DC 20555

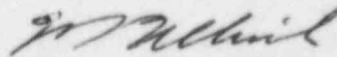
SUBJECT: Licensee Event Report
Peach Bottom Atomic Power Station - Unit 3

This LER deals with an isolated fuel bundle in the Unit 3 reactor core.

Reference:	Docket 50-278
Report Number:	3-85-12
Revision Number:	00
Event Date:	August 7, 1985
Report Date:	September 6, 1985
Facility:	Peach Bottom Atomic Power Station RD 1, Box 208, Delta, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator
Region I, USNRC
Mr. T. P. Johnson, Resident Inspector

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