

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Wolf Creek Generating Station										DOCKET NUMBER (2) 0 5 0 0 0 4 8 2 1 OF 0 3										PAGE (3) 1 OF 0 3																													
TITLE (4) Reactor Trip and Engineered Safety Features Actuation																																																	
EVENT DATE (5) 0 8 0 7 8 5 8 5										LER NUMBER (6) 0 6 0 0 0 0 9 0 4 8 5										REPORT DATE (7) 0 8 0 7 8 5										OTHER FACILITIES INVOLVED (8) 0 5 0 0 0 0																			
OPERATING MODE (9) 1										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)										DOCKET NUMBER(S) 0 5 0 0 0 0																													
POWER LEVEL (10) 0 9 1 2										20.402(b)										20.405(c)										50.73(a)(2)(iv)										73.71(b)									
										20.405(a)(1)(i)										50.36(c)(1)										50.73(a)(2)(v)										73.71(c)									
										20.405(a)(1)(ii)										50.36(c)(2)										50.73(a)(2)(vii)										OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
										20.405(a)(1)(iii)										50.73(a)(2)(i)										50.73(a)(2)(viii)(A)																			
										20.405(a)(1)(iv)										50.73(a)(2)(ii)										50.73(a)(2)(viii)(B)																			
										20.405(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(ix)																			

NAME Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services															TELEPHONE NUMBER 3 1 6 3 6 4 - 8 8 3 1														
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																													
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
SUPPLEMENTAL REPORT EXPECTED (14)															EXPECTED SUBMISSION DATE (15)														
YES (If yes, complete EXPECTED SUBMISSION DATE)															NO														

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At approximately 0626 CDT on August 7, 1985, a Reactor trip was initiated when the Main Turbine tripped due to a high-high level in a Moisture Separator Reheater (MSR). Following the Reactor trip, a Feedwater Isolation was initiated when Reactor Coolant System average temperature (Tavg) decreased to the low Tavg setpoint. Auxiliary Feedwater actuations and a Steam Generator Blowdown and Sample Isolation also occurred when Steam Generator water levels decreased to the low-low level setpoint. All required Engineered Safety Features and Reactor Protection System equipment functioned properly.

At the time of the event, the plant was in Mode 1, Power Operation, at approximately 92 percent reactor power. The plant was being operated on automatic controls and plant response throughout the event was normal.

No specific cause of the event has been identified although it was concluded that the source of the high-high level condition was a drain tank level control problem on MSR "A".

There was no damage to plant equipment or release of radioactivity as a result of this event, and at no time did conditions develop which could have posed a threat to the public health or safety.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Wolf Creek Generating Station	DOCKET NUMBER (2) 0 5 0 0 0 4 8 2 8 5 - 0 6 0 - 0 0 0 2 OF 0 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At approximately 0626 CDT on August 7, 1985, a Reactor trip was initiated when the Main Turbine [TA-TRB] tripped due to a high-high level in a Moisture Separator Reheater (MSR) drain tank [SN-TK]. Following the Reactor trip, a Feedwater Isolation was initiated when Reactor Coolant System (RCS)[AB] average temperature (Tavg) decreased to the low Tavg setpoint of 564 degrees F, and a motor-driven Auxiliary Feedwater Actuation and a Steam Generator Blowdown and Sample Isolation occurred when the water level in Steam Generator (S/G)[AB-SG] "C" decreased to the low-low level trip setpoint. A turbine-driven Auxiliary Feedwater Actuation also occurred when the water level in a second Steam Generator ("A") reached its low-low level trip setpoint. All required Engineered Safety Features and Reactor Protection System equipment functioned properly.

At the time of the event, the plant was in Mode 1, Power Operation, at approximately 92 percent reactor power during initial power ascension. The plant was being operated on automatic controls and plant response throughout the event was normal. Reactor Coolant System Tavg decreased to a minimum of 548 degrees F and Pressurizer [AB-PZR] level decreased from approximately 58 percent to approximately 22 percent. Steam Generator water levels decreased to a minimum of approximately 46 percent wide range indication and the Power Operated Relief valves [SB-RV] on Steam Generators "C" and "D" opened for approximately four and two minutes respectively. Actuated systems were restored to normal configurations per plant procedures and normal feedwater flow to the Steam Generators was restored at approximately 0734 CDT.

During the event, air-operated valve BM-HV-1, S/G "A" blowdown isolation [WI-ISV], and solenoid valve BM-HV-65, S/G "A" sample isolation [KN-ISV] responded properly by closing, but position indications on the Engineered Safety Features status panel did not agree with the position indication shown on the control boards. Subsequent investigation confirmed that the valves had closed and that the limit switches which indicate valve position were not properly adjusted. These switches have been readjusted to correctly indicate valve position.

Investigation into the cause of the Main Turbine trip determined that a Moisture Separator Reheater high-high level had initiated the trip. Checks of the MSR level controllers identified the need for some minor adjustments, which were accomplished, but did not identify a specific root cause for the event. It has been concluded that a MSR "A" drain tank level control problem caused the turbine trip. This conclusion was based on the absence of a MSR high level alarm in the Control Room prior to the trip. The high level alarm switch on MSR "A" drain tank was found misadjusted such that a high level condition in this tank would not be annunciated in the Control Room. This switch was adjusted and returned to operation.

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During the subsequent plant startup and power ascension, MSR system performance was closely monitored and fine tuned as appropriate. No specific problems were encountered and no high level conditions developed. There have been no previous similar events related to MSR level control.

There was no damage to plant equipment or release of radioactivity as a result of this event. At no time during the event did conditions develop that could have posed a threat to the health or safety of the public.



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

September 4, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Mr. R.P. Denise, Director
Division of Reactor Safety and Projects
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-215
Re: Docket No. STN 50-482
Subj: Licensee Event Report 85-060-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR
50.73 (a) (2) (iv) concerning an Engineered Safety Features
actuation.

Yours very truly,

Glenn L. Koester
Vice President - Nuclear

GLK:see

Enclosure

xc: PO'Connor (2), w/a
JCummins, w/a

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