NRC MONTHLY OPERATING REPORT

DOCKET NO. 50-528
UNIT NAME PVNGS-1
DATE 7/8/85
COMPLETED BY M. P. Richardson
TELEPHONE 602-932-5300
Ext. 6593

OPERATING STATUS

1. Unit Name: Palo Verde Nuclear Generating Station, Unit 1
2. Reporting Period: June 1985
3. Licensed Thermal Power (MWt): 3800
4. Nameplate Rating (Gross MWe): 1304
5. Design Electrical Rating (Net MWe): 1270
6. Maximum Dependable Capacity (Gross MWe): To be determined
7. Maximum Dependable Capacity (Net MWe): To be determined
8. If Changes Occur In Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: N/A
9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720	4344	4344
12. Number Of Hours Reactor Was Critical	695	839	839
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	375.52	375.52	375.52
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	328820.8	328820.8	328820.8
17. Gross Electrical Energy Generated (MWH)	60,200	60,200	60,200
18. Net Electrical Energy Generated (MWH)	16,292	16,292	16,292
19. Unit Service Factor	N/A	N/A	N/A
20. Unit Availability Factor	N/A	N/A	N/A
21. Unit Capacity Factor (Using MDC Net)	N/A	N/A	N/A
22. Unit Capacity Factor (Using DER Net)	N/A	N/A	N/A
23. Unit Forced Outage Rate	N/A	N/A	N/A

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
25. If Shutdown At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior To Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	5/85	5/25/85
INITIAL ELECTRICITY	6/85	6/10/85
COMMERCIAL OPERATION	11/85	N/A

AVERAGE DAILY UNIT POWER LEVEL

IE24

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MONTH: June 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	800
2	0	18	3700
3	0	19	3600
4	0	20	3000
5	0	21	4200
6	0	22	3800
7	0	23	4300
8	0	24	2400
9	0	25	1500
10	100	26	2800
11	500	27	4300
12	3100	28	4100
13	3300		4700
14	1800	30	6600
15	0		
16	200		

REFUELING INFORMATION

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1. Scheduled date for next refueling shutdown.

   03/01/87

2. Scheduled date for restart following refueling.

   04/19/87

3. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

   Not Yet Determined
   What will these be?

   Not Yet Determined

4. Scheduled date for submitting proposed licensing action and supporting information.

   Not Yet Determined

5. Important Licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   Not Yet Determined

6. The number of fuel assemblies.
   a) In the core. 241

   b) In the spent fuel storage pool. 0

7. Licensed spent fuel storage capacity. 1329

   Intended change in spent fuel storage capacity. None

8. Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.

   2002 (w/annual reloads and full core discharge capability).

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

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6/1 Unit in Mode 2

6/1 Continued with CEA worth measurements by exchange.

6/1 Completed low power physics testing.

6/2 Began CPC verification test and COLSS initialization.

6/5 Completed Mode change checklist for Mode 1, entered Mode 1.

6/7 1327 - Completed shell warming for the turbine, closed main steam stop valve #2 bypass.

6/8 Continued turbine generator startup.

6/8 1532 - Turbine at rated speed of 1800 rpm.

6/8 1650 - Turbine was at 1800 rpm for 1 hour and 20 minutes and then tripped on Hi vibration on the #8 bearing.

6/9 1150 - Turbine at rated speed of 1800 rpm.

6/9 1433 - Tripped Main Turbine manually due to Hi vibration on #7 bearing.

6/10 2226 - Approximately 20 min after the main generator was loaded CPC D DNBR started cycling causing a channel D trip. During the investigation, the main turbine had to be tripped on high vibration. When the turbine was unloaded and tripped, the DNBR cycling on CPC D stopped.

6/10 1149 - At set speed of 1800 rpm on Turbine.

6/10 1850 - Reactor power at 12%

6/10 2226 - Tripped Main Generator due to high vibration on bearing #7

6/11 0630 - Main turbine at 1800 rpm.

6/11 0651 - Tripped Main Turbine rather than having it spin unloaded during CPC testing.

6/12 1600 - Reactor at 20% power.

6/14 1156 - FWPT "B" tripped on low suction pressure, tripped Main Turbine, Reactor tripped on high Local Power Density (LPD). NOTE: On further investigation it was found the reactor trip was due to hi pressurizer pressure.

6/16 1323 - Criticality achieved on Unit 1 reactor.

6/17 Main turbine generator trip caused by oscillations in the feedwater control system. This caused a main feedwater pump trip.

6/18 1215 - Lowered turbine generator load down to 101 MW to cycle Atmospheric dump valve ADV SGA-184

6/18 1259 - Raised turbine generator load to 175 MW SGA-184 cycled 5 times (sat).

6/22 1611 - Increased surveillance frequency on CEA positions due to requirements of T.S. 4.1.3.6 with the PDIL auctioneer circuit inoperable.

6/23 Inserted Group 5 rods to 12" withdrawn for Rx Engineering in preparation for delta temperature and delta power testing.

6/24 1404 - Tripped turbine, began reducing power to 13% power
Note: The turbine shutdown was required because pressure point isolation valves MTN-V243 and MTN-V244 were blown off the main steam lines downstream of MTN-UV2002 causing a 1" hole in the steam line.

6/24 1610 - Rx power 11%.

6/25 0250 - Steam line break repaired, reset main turbine, commenced chest-warming.

6/25 2338 - Rx power 14.9%

6/26 1255 - Performed Automatic Economizer/Downcomer feedwater control valve transfer, transferred at 16.5% power. Initial test results are satisfactory.

6/27 Completed final economizer/downcomer valve testing, returned reactor power to 19.8% power.

6/27 1258 - Entered action statement on pressurizer safety valve position indication (3.3.3.6) for removal of temporary test equipment.

6/27 1430 - Exited action statement for pressurizer safety valve position indication.

6/28 RRS auto CEA insertion and withdrawal were tested satisfactory, (at low rate), following CEDMCS repairs.

6/29 2340 - Rx power 22%

6/29 0300 - Tavg dropped low due to inadvertant boration of the RCS. This boration was due to makeup mode selector switch in dilute position causing a boration vice dilution. Attempted to duplicate, problem could not be duplicated.

6/30 0356 - COLSS (CMC) declared operable

6/30 0430 - Began power ascension to 30% F.P.

6/30 2206 - Rx at 40% power generator load at 405 MWe.

UNIT SHUTDOWNS AND POWER REDUCTIONS

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TELEPHONE 932-5300
Ext 6593

No.	Date	Type [1]	Duration (Hours)	Reason [2]	Method of Shutting Down Reactor [3]	LER No.	System Code [4]	Component Code [5]	Cause & Corrective Action to Prevent Recurrence
1	6/14/85	F	25	A	3	85-019	HBG	Valve	See Attached

[1] F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3] Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation from Previous Month
5-Reduction of 20% or Greater in the Past 24 Hours
6-Other (Explain)

[4] Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

[5] Exhibit H-Same Source

2940u

The "A" Main Feedwater Pump Mini-Flow Control Valve did not control properly, allowing excessive condensate flow back to the condenser. This in conjunction with improper operation of Condensate Pump Mini-Flow Control Valves caused a low suction pressure trip of the "B" Main Feedwater Pump at 1155. The reactor operators manually tripped the turbine-generator, started auxiliary feedwater and began manually reducing reactor power by insertion of CEA's attempting to reduce power to within the capability of the Auxiliary feedwater System. At 1156 the reactor tripped on high pressurizer pressure.

Cause of the trip was improper operation of the "A" Main Feedwater Pump Mini-Flow Control Valve.

Z

Arizona Nuclear Power Project
P.O. BOX 52034 • PHOENIX, ARIZONA 85072-2034

Learned W. Barry, Director
Office of Resource Management
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

ANPP-33016-EEVB/GEC
July 15, 1985

Subject: Palo Verde Nuclear Generating Station (PVNGS)
Unit 1
Docket No. STN 50-528, License No. NPF-41
June Monthly Operating Report
File: 85-056-026; G.1.01.10

Dear Mr. Barry:

Attached please find the June 1985 Monthly Operating Report prepared and submitted pursuant to Specification 6.9.1.6 of Appendix A (Technical Specifications) to the Palo Verde Nuclear Generating Station, Unit 1 Operating License. By copy of this letter we are also forwarding a copy of the Monthly Operating Report to the Regional Administrator of the Region V Office.

If you have any questions or concerns, pleae contact me.

Very truly yours,

EE Van Brunt/DBK

E. E. Van Brunt, Jr.
Executive Vice President
Project Director

EEVBJr/GEC/s:h
Attachments

cc: J. B. Martin (all w/a)
R. P. Zimmerman
E. A. Licitra
A. C. Gehr

IE24
1/1