

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JUNE 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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1. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984, and commercial power operation was commenced on June 19, 1984.

This report was compiled by Richard J. Rohrer, telephone number (615)357-6761 extension 575.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

JUNE 1-30

The reactor was subcritical, and the generator was off-line for the entire month of JUNE. Unit Two is in a scheduled outage for maintenance, testing, and modifications.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED
MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

Amendment 12 was made to the unit two Technical Specifications, incorporating a revision to the system response time specification for Main Steam Line Low Pressure Isolation.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L38868	RHR "B" & "C" Area Fan Motor.	Faulty Bearings	Could not lubricate Motor.	Replaced bearings.
L45362	Suppression Pool Narrow Range Level Recorder.	Leaking Equalizing Valve and Out-of-tolerance Instrument.	False indications.	Cleaned equalizing valve and recalibrated instruments.
L47064	"A" RHR Pump Minimum Flow Valve.	Packing leaked.	Unwanted leakage.	Replaced packing.
L47218	Outboard Feedwater check valve "B".	Leaky Solenoid Valves.	Check Valve could not be held in the closed position.	Tested and replaced solenoid valves.
L47284	RHR "A" Service Water Pump.	Oil needed to be changed.	Oil leak in bearing.	Changed oil.
L48162	Diesel Generator "A" watt meter.	Out-of-tolerance.	Local indication was incorrect.	Recalibrated meter.
L48299	RHR Pump Minimum bypass flow switch.	Jammed Equalizing valve on 5-valve-manifold.	Could not calibrate instrument.	Replaced Equalizing valve stem.
L48925	Average Power Range Monitor F.	Blown Fuse.	Half of downscale lights were out.	Replaced fuse.
L49040	HPCS Pump Room Cubicle Fan.	Faulty plunger interlock on HPCS Breaker.	Fan did not autostart when HPCS started.	Replaced plunger interlock.
L49047	Circuit Breaker for valve 2B21-P071.	Broken Reset Button on thermal overload device.	Could not reset breaker.	Replaced thermal overload device.
L49049	Circuit Breaker for valve 2B21-P073.	Faulty thermal overload device.	Could not reset breaker.	Replaced thermal overload device.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L49078	RHR Shutdown Cooling Outboard Isolation Valve.	Loose set screws in gears.	Local valve indication always at 100% open.	Adjusted indicator and tightened set screws.
L49106	HPCS Pump discharge flow meter.	Out-of-tolerance.	Indicated pump discharge flow was too low.	Recalibrated flow meter.
L49117	RHR "A" Minimum Flow Valve Flow Switch.	Out-of-Tolerance.	Minimum flow valve did not open when it should have.	Recalibrated switch.
L49419	RCIC steam line drain valve.	Faulty spring and cracked retaining ring.	Valve leaked.	Replaced spring and retaining ring.
L49428	Scram discharge Volume isolation valve.	Mispositioned actuating arm.	Incorrect position indication.	Adjusted and tightened actuating arm.
L49443	Reactor Vessel Level Switch.	High and Low side sensing lines installed on the wrong ports of the instrument.	Instrument inoperable.	Rerouted sensing lines to the correct ports.
L49541	Reactor Building HVAC supply damper.	Broken limit switch.	Limit switch would not pick up, thus preventing supply fans from starting.	Repaired limit switch.
L49765	"B" Outboard MSIV.	Broken limit switch.	Improper position indication.	Replaced limit switch.
L49801	"C" Inboard MSIV.	Broken limit switch.	Improper position indication.	Replaced limit switch.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, June 1 through June 30, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-023-00	5-23-85	Spurious Safety Relief Valve Actuation.
85-024-00	5-31-85	Group I Isolation from Low Condenser Vacuum.
85-025-00	5-6-85	Reactor Scrams from LES-RP-02.
85-026-00	5-22-85	Diesel Generator Start Failure.
85-027-00	5-27-85	HPCS CY Return Line Rupture.
85-028-00	6-4-85	CRD Low Header Pressure Scram.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES</u> <u>ACTUATED</u>	<u>NO & TYPE</u> <u>ACTUATIONS</u>	<u>PLANT</u> <u>CONDITION</u>	<u>DESCRIPTION</u> <u>OF EVENT</u>
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There were no safety relief valves operated during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-834-85	HPCS Pump Suction Valves.	Environmental Qualification Modification. (EQ Mod).
2-841-85	B/C RHR Cubicle Cooler.	Replace Bearings.
2-842-85	"B" RHR Pump.	Repair puffer piston.
2-843-85	Valve 2E12-F051B	Adjust Valve.
2-844-85	Division II RHR Service Water Process Radiation Monitor.	Relocate Suction.
2-845-85	"B" RHR Service Water Strainer.	Repair leak.
2-846-85	"B" RHR Shutdown Cooling Testable Check Valve.	Repack Valve.
2-847-85	2DO03P.	Testing.
2-855-85	"A" Diesel Generator.	Equipment Protection.
2-860-85	HPCS Suction from Cycled condensate and full flow test valve.	Isolate pipe break.
2-861-85	"B" Diesel Generator Air compressor.	Maintenance.
2-866-85	"B" and "C" RHR	Prevent injection to reactor vessel.
2-869-85	"B" RHR Service Water.	Hydrostatic Test.
2-871-85	"B" Diesel Generator.	Repair Local and Control Room Meters.
2-873-85	"B" RHR Service Water Process Radiation Monitor.	Modification.
2-874-85	RCIC Drain Valve.	Repair weld.
2-879-85	RCIC Drain Valve.	Repair weld.
2-884-85	2E51-F063.	Inspect Limit Switches.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-886-85	HPCS Pump.	Install Bracket.
2-892-85	"B" Diesel Generator Immersion Heater.	Replace Transformer.
2-899-85	RCIC Turbine.	Change Oil.
2-900-85	RCIC Turbine.	Flush Oil.
2-912-85	2E51-F063.	Inspect Limit Switches.
2-920-85	2E12-F053A.	Replace Motor.
2-931-85	4.16 KV feed to switch gear 237X and 237Y.	Equipment Protection.
2-956-85	HPCS Pump Breaker.	Repair Auxiliary Contactor.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculation manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the radioactive waste treatment system during this reporting period.

1. OPERATING DATA REPORT

DOCKET NO. 050-374

UNIT LaSalle Two

DATE July 10, 1985

COMPLETED BY Richard J. Rohrer

TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: June, 1985 GROSS HOURS IN REPORTING PERIOD: 720
 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY): N/A
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|--|------------|------------|------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | 0.0 | 1399.8 | 3011.6 |
| 6. REACTOR RESERVE SHUTDOWN HOURS | 0.0 | 0.0 | 125.3 |
| 7. HOURS GENERATOR ON LINE | 0.0 | 1397.3 | 2934.7 |
| 8. UNIT RESERVE SHUTDOWN HOURS | 0.0 | 0.0 | 0.0 |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | 0.0 | 4387385 | 8894977 |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | 0.0 | 1460387 | 2945373 |
| 11. NET ELEC. ENERGY GENERATED (MWH) | -8557 | 1373636 | 2765953 |
| 12. REACTOR SERVICE FACTOR | 0.0% | 32.1% | 49.0% |
| 13. REACTOR AVAILABILITY FACTOR | 0.0% | 32.1% | 51.1% |
| 14. UNIT SERVICE FACTOR | 0.0% | 32.0% | 47.8% |
| 15. UNIT AVAILABILITY FACTOR | 0.0% | 32.0% | 47.8% |
| 16. UNIT CAPACITY FACTOR (USING MDC) | -1.1% | 30.4% | 43.5% |
| 17. UNIT CAPACITY FACTOR(USING DESIGN MWe) | -1.1% | 29.2% | 41.8% |
| 18. UNIT FORCED OUTAGE RATE | 0.0% | 0.0% | 3.9% |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
An outage for maintenance and surveillance was begun at 0520 on February 28, 1985.
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP July 12, 1985

ATTACHMENT B
3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE JULY 10, 1985
COMPLETED BY Richard J. Rohrer
TELEPHONE (815) 357-6761

REPORT MONTH JUNE 1985

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
3	85-0228	S	720.00	B	2	Maintenance and Surveillance Outage began 2-28-85 continues

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: July 10, 1985
COMPLETED BY: Richard J. Rohrer
TELEPHONE: (815) 357-6761
MONTH: June 1985

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

1.	-5
2.	-6
3.	-5
4.	-5
5.	-7
6.	-9
7.	-6
8.	-6
9.	-7
10.	-14
11.	-15
12.	-14
13.	-14
14.	-14
15.	-13
16.	-14

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

17.	-13
18.	-13
19.	-14
20.	-15
21.	-15
22.	-14
23.	-14
24.	-14
25.	-15
26.	-14
27.	-14
28.	-15
29.	-15
30.	-16