

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346  
 UNIT Davis-Besse Unit 1  
 DATE July 10, 1985  
 COMPLETED BY Morteza Khazrai  
 TELEPHONE (419) 249-5000,  
Ext. 290

MONTH June 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	786
2	166
3	0
4	108
5	389
6	459
7	762
8	779
9	37
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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# OPERATING DATA REPORT

DOCKET NO. 50-346  
 DATE July 10, 1985  
 COMPLETED BY Morteza Khazrai  
 TELEPHONE (419) 249-5000,  
 Ext. 290

## OPERATING STATUS

1. Unit Name: Davis-Besse Unit 1
2. Reporting Period: June 1985
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 915
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 904
7. Maximum Dependable Capacity (Net MWe): 860

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720	4,343.0	60,648.0
12. Number Of Hours Reactor Was Critical	148.8	2,845.6	35,877.1
13. Reactor Reserve Shutdown Hours	44.7	44.7	4,058.8
14. Hours Generator On-Line	140.5	2,730.5	34,371.8
15. Unit Reserve Shutdown Hours	0.0	0.0	1,732.5
16. Gross Thermal Energy Generated (MWH)	279,418	6,312,178	81,297,600
17. Gross Electrical Energy Generated (MWH)	90,582	2,087,278	26,933,622
18. Net Electrical Energy Generated (MWH)	81,419	1,942,921	25,233,177
19. Unit Service Factor	19.5	62.9	56.7
20. Unit Availability Factor	19.5	62.9	59.5
21. Unit Capacity Factor (Using MDC Net)	13.1	52.0	48.4
22. Unit Capacity Factor (Using DER Net)	12.5	49.4	45.9
23. Unit Forced Outage Rate	80.5	18.3	17.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY

INITIAL ELECTRICITY

COMMERCIAL OPERATION

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June 1985

DOCKET NO. 50-346  
 UNIT NAME Davis-Besse Unit 1  
 DATE July 10, 1985  
 COMPLETED BY Morteza Khazrai  
 TELEPHONE 419-249-5000, Ext. 290

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
6	85 06 02	F	53.0	A	3	85-011	SB	XCV	The main turbine tripped during main turbine control valve testing. The Anticipatory Reactor Trip System (ARTS) tripped the reactor.
7	85 06 09	F	526.5	A	3	85-013	JK	SC	The reactor tripped on high Reactor Coolant System (RCS) pressure when No. 1 Main Feed Pump developed control problems and tripped on overspeed.  (See Operational Summary for further details)

<sup>1</sup> F: Forced  
S: Scheduled

<sup>2</sup> Reason:  
A-Equipment Failure (Explain)  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain)  
H-Other (Explain)

<sup>3</sup> Method:  
1-Manual  
2-Manual Scram  
3-Automatic Scram  
4-Continuation from  
Previous Month  
5-Load Reduction  
9-Other (Explain)

<sup>4</sup> Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File  
(NUREG-0161)

<sup>5</sup> Exhibit I - Same Source

REFUELING INFORMATION

DATE: June 1985

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling shutdown: Spring, 1986
3. Scheduled date for restart following refueling: Summer, 1986
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Ans: Expect the Reload Report to require standard reload fuel design Technical Specification changes (3/4.1 Reactivity Control Systems and 3/4.2 Power Distribution Limits).

5. Scheduled date(s) for submitting proposed licensing action and supporting information: Winter, 1985
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Ans: None identified to date.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 177                      (b) 204 - Spent Fuel Assemblies

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735                      Increase size by: 0 (zero)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1992 - assuming ability to unload the entire core into the spent fuel pool is maintained.

OPERATIONAL SUMMARY  
June, 1985

6/1/85 - 6/4/85

Reactor power was maintained at approximately 90% power (power was limited to 90% due to the noise in the Reactor Coolant System flow transmitters which is believed to have caused the reactor trip on April 24, 1985) until 0400 hours on June 2, 1985, when a manual power reduction was initiated to perform turbine valve testing. During main turbine control valve testing, a turbine trip occurred. The Anticipatory Reactor Trip System tripped the reactor. The root cause of the trip was the turbine tripping on high vibration when the #3 Turbine Control Valve was reopened.

The reactor was critical at 0247 hours on June 4, 1985. The turbine-generator was synchronized on line at 0135 hours on June 4, 1985.

6/5/85 - 6/8/85

Reactor power was increased to approximately 50% power which was achieved on June 5, 1985. Reactor power was limited at this power level to troubleshoot the main feed pump control oil pressure problems.

After the completion of troubleshooting on #1 Main Feed Pump Turbine, reactor power was increased to approximately 90% which was attained on June 7, 1985.

6/9/85 - 6/30/85

Reactor power was maintained at approximately 90% power until 0135 hours on June 9, 1985, when a reactor trip occurred. This transient was initiated when the #1 Main Feed Pump developed control problems and tripped on overspeed. The reactor tripped on high Reactor Coolant System pressure due to inadequate feedwater being supplied from the #2 Main Feed Pump during the plant runback. See Licensee Event Report 85-013 for further details.

The plant was cooled down and remained shutdown the remainder of the month to investigate the event.



July 10, 1985

Log No. K85-1016  
File: RR 2 (P-6-85-06)

Docket No. 50-346  
License No. NPF-3

Mr. Norman Haller, Director  
Office of Management and Program Analysis  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Mr. Haller:

Monthly Operating Report, June 1985  
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit 1 for the month of June 1985.

If you have any questions, please feel free to contact Morteza Khazrai at (419) 249-5000, Extension 290.

Yours truly,

Stephen M. Quennoz  
Plant Manager  
Davis-Besse Nuclear Power Station

SMQ/MK/ljk

Enclosures

cc: Mr. James G. Keppler, w/1  
Regional Administrator, Region III

Mr. James M. Taylor, Director, w/2  
Office of Inspection and Enforcement

Mr. Walt Rogers, w/1  
NRC Resident Inspector

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