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August 5, 1985
RBG - 21774
File No. G9.5

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Denton:

River Bend Station - Unit 1
Docket No. 50-458

Enclosed for your review is Gulf States Utilities Company proposed revisions to the River Bend Station Technical Specifications. These changes as identified in Enclosure 1 conform to the information contained in the Final Safety Analysis Report and the Safety Evaluation Report. Enclosure 2 contains the proposed page markups from the "FINAL DRAFT."

Sincerely,

J. E. Booker

J. E. Booker
Manager-Engineering
Nuclear Fuels & Licensing
River Bend Nuclear Group

JEB/WJR/JEP

Enclosure (2)



A001
1/1

ENCLOSURE 1

| Page Number | Description/Justification |
| --- | --- |
| 3/4 3-57 | Change the allowable value from " $\leq$ 55 inches " to " $\leq$ 52 inches" for the Reactor Vessel Water Level High Level 8. This change is as the result of the installation of a new Rosemount transmitter and the corresponding change to the design specification data sheet. The trip setpoint change was provided in GSU letter July 19, 1985 (RBG-21572). |
| 3/4 3-62 and 3/4 6-63 | Section 3/4.6.6.2 (Primary/Containment/Drywell Mixing System) The references to "valve" should be changed to read "flow path" as there is actually more than one valve in each line. (See FSAR Figure 6.2-66) |
| 5-5 | Change the average enrichment reference in Section 5.3 from "1.70" to "1.85". GSU provided in a letter dated May 22, 1985 (RBG-21090) a discussion pertaining to the Control Cell Core design which is to be implemented for the initial core load. The average enrichment value is documented in NEDE-30854. |

ENCLOSURE 2

TABLE 3.3.5-2

REACTOR CORE ISOLATION COOLING SYSTEM ACTUATION INSTRUMENTATION SETPOINTS

| FUNCTIONAL UNITS | TRIP SETPOINT | ALLOWABLE VALUE |
|---|---|---|
| 1. Reactor Vessel Water Level - Low Low Level 2 | $\geq$ -43 inches* | $\geq$ -47 inches |
| 2. Reactor Vessel Water Level - High Level 8 | $\leq$ 51 inches* | $\leq$ ~~55~~ 52 inches |
| 3. Condensate Storage Tank Level - Low | $\geq$ 0 inches | $\geq$ -4.5 inches |
| 4. Suppression Pool Water Level - High | $\leq$ 6.5 inches | $\leq$ 8 inches |
| 5. Manual Initiation | NA | NA |

*See Bases Figure B 3/4 3-1.

PRIMARY CONTAINMENT/DRYWELL HYDROGEN MIXING SYSTEM

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LIMITING CONDITION FOR OPERATION

3.6.6.2 Two primary containment/drywell hydrogen mixing systems shall be OPERABLE and the inlet and outlet valves shall be closed, except one inlet or outlet (6-inch valve) may be opened for controlling drywell pressure with the following time limits: (flow path)

a. In OPERATIONAL CONDITION 3, not to exceed 90 hours per 365 days, and

b. In OPERATIONAL CONDITION 1 or 2, not to exceed 5 hours per 365 days.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

a. With one hydrogen mixing system inlet or outlet ~~valve~~ (flow path) open during OPERATIONAL CONDITIONS 1 and 2 for more than 5 hours per 365 days, immediately close the hydrogen mixing valves or be in at least HOT SHUTDOWN within the next 12 hours.

b. With one hydrogen mixing system inlet or outlet ~~valve~~ (flow path) open during OPERATIONAL CONDITION 3 for more than 90 hours per 365 days, immediately close the hydrogen mixing valves or be in at least COLD SHUTDOWN within the next 24 hours.

c. With one primary containment/drywell hydrogen mixing system inoperable, restore the inoperable system to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours.

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4.6.6.2 Each primary containment/drywell hydrogen mixing system shall be demonstrated OPERABLE:

a. At least once per 7 days, by determining the cumulative time that:

1. The hydrogen mixing system inlet or outlet ~~valve~~ (flow path) has been open during OPERATIONAL CONDITIONS 1 and 2 during the past 365 days, and

2. The hydrogen mixing system inlet or outlet ~~valve~~ (flow path) has been open during OPERATIONAL CONDITION 3 during the past 365 days.

b. During each COLD SHUTDOWN, if not performed within the previous 92 days, by:

1. Starting the system from the control room, and

2. Verifying that the system operates for at least 15 minutes.

c. At least once per 18 months by verifying a system flow rate of at least 600 cfm.

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## 5.3 REACTOR CORE

### FUEL ASSEMBLIES

5.3.1 The reactor core shall contain 624 fuel assemblies. Each fuel assembly shall contain 62 fuel rods and two water rods, all clad with Zircaloy-2. Each fuel rod shall have a nominal active fuel length of 150 inches. The initial core loading shall have a maximum average enrichment of ~~1.70~~ 1.85 weight percent U-235. Reload fuel shall be similar in physical design to the initial core loading.

### CONTROL ROD ASSEMBLIES

5.3.2 The reactor core shall contain 145 control rod assemblies, each consisting of a cruciform array of stainless steel tubes surrounded by a cruciform shaped stainless steel sheath. Each tube shall contain 143.7 inches of boron carbide ($B_4C$) powder.

## 5.4 REACTOR COOLANT SYSTEM

### DESIGN PRESSURE AND TEMPERATURE

5.4.1 The reactor coolant system is designed and shall be maintained:

a. In accordance with the code requirements specified in Section 5.2 of the FSAR, with allowance for normal degradation pursuant to the applicable Surveillance Requirements,

b. For a pressure of:

1. 1250 psig on the suction side of the recirculation pump.

2. 1650 psig from the recirculation pump discharge to the outlet side of the discharge shutoff valve.

3. 1550 psig from the discharge shutoff valve to the jet pumps.

c. For a temperature of 575°F.

### VOLUME

5.4.2 The total water and steam volume of the reactor vessel and recirculation system is approximately 16,000 cubic feet.

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