



GULF STATES UTILITIES COMPANY

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August 7, 1985
RBG-21815
File No. G9.5

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Denton:

River Bend Station-Unit 1
Docket No. 50-458

As per discussions with your Staff and at your Staff's request, Gulf States Utilities Company (GSU) provides in Enclosure 1 proposed revisions to the River Bend Station Technical Specifications. These revisions are changes which have been previously discussed and reviewed by your Staff and are considered by GSU to be enhancements to the "FINAL DRAFT". It is our understanding with your Staff that these minor changes will not impact the issuance of the Operating License.

Sincerely,

J. E. Booker
Manager-Engineering,
Nuclear Fuels & Licensing
River Bend Nuclear Group

JEB/ERG/JEP

Enclosure (1)

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ENCLOSURE 1

TABLE 3.3.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION

TABLE NOTATIONS

- (a) A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the tripped condition provided at least one OPERABLE channel in the same trip system is monitoring that parameter.
- (b) The "shorting links" shall be removed from the RPS circuitry prior to and during the time any control rod is withdrawn*, and ~~shutdown margin demonstrations are being performed per Specification 3.10.3.~~
- (c) An APRM channel is inoperable if there are less than 2 LPRM inputs per level or less than 11 LPRM inputs to an APRM channel.
- (d) This function is not required to be OPERABLE when the reactor pressure vessel head is removed per Specification 3.10.1.
- (e) This function shall be automatically bypassed when the reactor mode switch is not in the Run position.
- (f) This function is not required to be OPERABLE when DRYWELL INTEGRITY is not required.
- (g) With any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.
- (h) This function shall be automatically bypassed when turbine first stage pressure is ≤ 187 psig,** equivalent to THERMAL POWER less than 40% of RATED THERMAL POWER.

Unless adequate shutdown margin has been demonstrated per Specification 3.1.1,

*Not required for control rods removed per Specification 3.9.10.1 or 3.9.10.2.

**To allow for instrumentation accuracy, calibration and drift, a setpoint of ≤ 177 psig turbine first stage pressure shall be used.

TABLE 3.3.6-1 (Continued)
CONTROL ROD BLOCK INSTRUMENTATION

FINAL DRAFT

ACTION

- ACTION 60 - Declare the RPCS inoperable and take the ACTION required by Specification 3.1.4.2.
- ACTION 61 - With the number of OPERABLE Channels:
- a. One less than required by the Minimum OPERABLE Channels per Trip Function requirement, restore the inoperable channel to OPERABLE status within 7 days or place the inoperable channel in the tripped condition within the next hour. #
 - b. Two or more less than required by the Minimum OPERABLE Channels per Trip Function requirement, place at least one inoperable channel in the tripped condition within one hour. #
- ACTION 62 - With the number of OPERABLE channels less than required by the Minimum OPERABLE Channels per Trip Function requirement, place the inoperable channel in the tripped condition within one hour. #

NOTES

* With more than one control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.

** OPERABLE channels must be associated with SRM required OPERABLE per Specification 3.9.2.

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- (a) This function shall be automatically bypassed if detector count rate is ≥ 100 cps or the IRM channels are on range 3 or higher.
 - (b) This function shall be automatically bypassed when the associated IRM channels are on range 8 or higher.
 - (c) This function shall be automatically bypassed when the IRM channels are on range 3 or higher.
 - (d) This function shall be automatically bypassed when the IRM channels are on range 1.

The provisions of Specification 3.0.4 are not applicable

TABLE NOTATION

- ACTION 100 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases may continue for up to 14 days provided that prior to initiating a release:
- At least two independent samples are analyzed in accordance with Specification 4.11.1.1.1 and
 - At least two technically qualified members of the facility staff independently verify the release rate calculations and discharge line valving;
- Otherwise, suspend release of radioactive effluents via this pathway.
- ACTION 101 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided that, at least once per 12 hours, grab samples are collected and analyzed for gross radioactivity (beta or gamma) at a limit of detection of at least 10^{-7} microcuries/ml.
- ACTION 102 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided the flow rate is estimated at least once per 4 hours during actual releases. Pump curves generated in situ may be used to estimate flow.
- Otherwise, suspend release of radioactive effluents via this pathway.

TABLE NOTATION

* At all times.

** During main condenser offgas treatment system operation.

*** During operation of the main condenser air ejector.

ACTION 121 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided the flow rate is estimated at least once per 4 hours.

ACTION 122 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided grab samples are taken at least once per 12 hours and these samples are analyzed for gross activity within 24 hours.

ACTION 123 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, operation of main condenser offgas treatment system may continue for up to 30 days provided grab samples are collected at least once per 4 hours and analyzed within the following 4 hours.

ACTION 124 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided samples are continuously collected with auxiliary sampling equipment as required in Table 4.11.2.1.2-1. that within 8 hours

Otherwise, suspend release of radioactive effluents via this pathway.

3/4.3.8 TURBINE OVERSPEED PROTECTION SYSTEM

LIMITING CONDITION FOR OPERATION

3.3.8 At least one turbine overspeed protection system shall be OPERABLE.

APPLICABILITY: OPERATIONAL CONDITIONS 1 and 2.

ACTION:

- a. With one turbine control valve or one turbine stop valve per high pressure turbine steam lead inoperable and/or with one turbine intercept valve or one turbine intermediate stop valve per low pressure turbine steam lead inoperable, restore the inoperable valve(s) to OPERABLE status within 72 hours or close at least one valve in the affected steam lead or isolate the turbine from the steam supply within the next 6 hours.
- b. With the above required turbine overspeed protection system otherwise inoperable, within 6 hours isolate the turbine from the steam supply.
- c. *The provisions of Specification 3.0.4 are not applicable.*

SURVEILLANCE REQUIREMENTS

4.3.8.1 The provisions of Specification 4.0.4 are not applicable.

4.3.8.2 The above required turbine overspeed protection system shall be demonstrated OPERABLE:

- a. At least once per 7 days by cycling each of the following valves through at least one complete cycle from the running position:
 - 1) Four high pressure turbine stop valves,
 - 2) Four low pressure turbine intermediate stop valves,
 - 3) Four high pressure turbine control valves, and
 - 4) Four low pressure turbine intercept valves.
- b. At least once per 18 months by performance of a CHANNEL CALIBRATION of the turbine overspeed protection system.
- c. At least once per 40 months by disassembling at least one of each of the above valves and performing a visual and surface inspection of all valve seats, disks and stems and verifying no unacceptable flaws or excessive corrosion. If unacceptable flaws or excessive corrosion are found, all other valves of that type shall be inspected.

CONTAINMENT SYSTEMS

FINAL DRAFT

3/4.6.3 DEPRESSURIZATION SYSTEMS

SUPPRESSION POOL

LIMITING CONDITION FOR OPERATION

3.6.3.1 The suppression pool shall be OPERABLE with the pool water:

- a. Volume between 137,571 ft³ and 141,036 ft³, equivalent to a level between 19'6" and 20'0" and a
- b. Maximum average temperature of 95°F during OPERATIONAL CONDITION 1 or 2, except that the maximum average temperature may be permitted to increase to:
 1. 105°F during testing which adds heat to the suppression pool.
 2. 110°F with THERMAL POWER less than or equal to 1% of RATED THERMAL POWER.
 3. ~~120°F with the main steam line isolation valves closed following a scram.~~

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, and 3.

ACTION:

- a. With the suppression pool water level outside the above limits, restore the water level to within the limits within 1 hour or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. ~~In OPERATIONAL CONDITION 1 or 2,~~ With the suppression pool average water temperature greater than 95°F, restore the average temperature to less than or equal to 95°F within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours, except, as permitted above:
 1. With the suppression pool average water temperature greater than 105°F during testing which adds heat to the suppression pool, stop all testing which adds heat to the suppression pool and restore the average temperature to less than 95°F within 24 hours or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- c. Maximum average temperature of 95°F during OPERATIONAL CONDITION 3, except that the maximum average temperature may be permitted to increase to 120°F with the main steam line isolation valves closed following a scram.

SURVEILLANCE REQUIREMENTS (Continued)

2. At least once per hour, when suppression pool average water temperature is greater than or equal to 95°F, by verifying suppression pool average water temperature to be less than or equal to 110°F and THERMAL POWER to be less than or equal to 1% of RATED THERMAL POWER.

- c. ³ [At least once per 30 minutes ^{IN OPERATIONAL CONDITION 3} following a scram, with suppression pool average water temperature greater than or equal to 95°F, by verifying suppression pool average water temperature less than or equal to 120°F.]

PLANT SYSTEMS

3/4.7.10 STRUCTURAL SETTLEMENT

LIMITING CONDITION FOR OPERATION

3.7.10 Structural settlement shall be within the predicted values as shown in Table 3.7.10-1 and calculated differential settlements shall be within the allowable ranges shown in Table 3.7.10-2 for the following structures:

- a. Reactor Building
- b. Auxiliary Building
- c. Fuel Building
- d. Control Building
- e. Diesel Generator Building
- f. Standby Cooling Tower, Basin and Pump House
- g. BF Tunnel
- h. Main Steam Tunnel
- i. E Tunnel
- j. G Tunnel

APPLICABILITY: At all times.

ACTION:

With the measured structural settlement of any of the above required structures outside of the limits of Tables 3.7.10-1 and 3.7.10-2, prepare and submit, within the next 30 days, a Special Report to the Commission, pursuant to Specification 6.9.2, providing a record of the settlement measurements and the predicted settlement, an analysis to demonstrate the continued structural integrity of the affected structure(s), and plans to monitor the settlement of the affected structure(s) in the future. The provisions of Specification 3.0.3 and 3.0.4 are not applicable to the limits established in Table 3.7.10-1.

SURVEILLANCE REQUIREMENTS

4.7.10 The structural settlement of the above required structures shall be demonstrated to be within the limits of Tables 3.7.10-1 and 3.7.10-2:

- a. At least once per 92 days, until there is essentially no movement during those 92 days.
- b. At least once per 24 months, for at least 10 years.
- c. Following any seismic event equal to or greater than an Operational Basis Earthquake (OBE).

SURVEILLANCE REQUIREMENTS (Continued)

- b. Performance of a CHANNEL FUNCTIONAL TEST:
 - 1. Within 24 hours prior to the start of CORE ALTERATIONS, and
 - 2. At least once per 7 days.
- c. Verifying that the channel count rate is at least 0.7 cps*:
 - 1. Prior to control rod withdrawal,
 - 2. Prior to and at least once per 12 hours during CORE ALTERATIONS, and
 - 3. At least once per 24 hours, except that:
 - a. During fuel unloading, the required count rate may be permitted to be less than 0.7 cps*.
 - b. Prior to and during fuel loading, until sufficient fuel has been loaded to maintain at least 0.7 cps*, the required count rate may be achieved by:
 - a) Use of portable external source, or
 - b) Loading up to 2 fuel assemblies in cells containing inserted control rods around an SRM.
- d. Verifying within 8 hours prior to and at least once per 12 hours during ~~the~~
 - ① ~~the time any control rod is withdrawn,~~ ## or
 - 2. ~~Shutdown margin demonstrations,~~

that the RPS circuitry "shorting links" have been removed unless adequate shutdown margin has been demonstrated per Specification 3.1.1.

*Provided signal to noise ratio ≥ 2 , otherwise use 3.0 cps.

Not required for control rods removed per Specification 3.9.10.1 or 3.9.10.2.

RADIOACTIVE EFFLUENTS

MAIN CONDENSER

FINAL DRAFT

LIMITING CONDITION FOR OPERATION

3.11.2.7 The release rate of the sum of the activities from the noble gases* measured prior to the holdup pipe shall be limited to less than or equal to 290 millicuries/sec after 30 minutes decay.

APPLICABILITY: Whenever the main condenser offgas treatment system is in operation.

ACTION:

With the release rate of the sum of the activities from the noble gases* prior to the holdup pipe exceeding 290 millicuries/sec after 30 minutes decay, restore release rate to within its limit within 72 hours or be in at least HOT SHUTDOWN within the next 12 hours.

SURVEILLANCE REQUIREMENTS

4.11.2.7.1 The radioactivity of noble gases prior to the holdup pipe shall be continuously monitored in accordance with Specification 3.3.7.1.

4.11.2.7.2 The release rate of the sum of the activities from the noble gases* measured prior to the holdup pipe shall be determined, at the following frequencies, to be within the limits of Specification 3.11.2.7 by performing an isotopic analysis of a representative sample of gases taken prior to the holdup pipe.

- a. At least once per 31 days.
- b. Within 4 hours following an increase, as indicated by the Noble Gas Activity Monitor, of greater than 50%, after factoring out increases due to changes in THERMAL POWER level, in the nominal steady state fission gas release from the primary coolant.
- c. The provisions of Specification 4.0.4 are not applicable

*Gamma scintillation detectors are used to measure the Kr-85m, -87, -88 and Xe-133, -133m, -135, -138 contribution after 30 minutes decay.

JUL 19 1985