

ATTACHMENT (3)

UNIT 1

MARKED-UP TECHNICAL SPECIFICATION

PAGES

3/4 6-1 -- 3/4 6-5

3/4 9-10

6-7

B 3/4 6-1

9612030273 961126
PDR ADOCK 05000317
P PDR

**Baltimore Gas and Electric Company
License Amendment Request
November 26, 1996**

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.*

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION: Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

- a. At least once per 31 days by verifying that all penetrations** not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except for valves that are open under administrative control as permitted by Specification 3.6.4.1.
- b. By verifying that each containment air lock is in compliance with the requirements of Specification 3.6.1.3.
- c. By verifying that the equipment hatch is closed and sealed, prior to entering MODE 4 following a shutdown where the equipment hatch was opened, by conducting a Type B test per 10 CFR Part 50, Appendix J.

in accordance with the Containment Leakage Rate Testing Program.

* Hydrogen purge containment vent isolation valves shall be opened for containment pressure control, airborne radioactivity control, and surveillance testing purposes only.

The shutdown cooling isolation valves may be opened when the RCS temperature is below 300°F to establish shutdown cooling flow.

** Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked, sealed, or otherwise secured in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days.

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

Containment Leakage

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. A maximum allowable containment leakage rate, L_a , as specified in Specification 6.5.6, "Containment Leakage Rate Testing Program."

b. A combined leakage rate of $< 0.50 L_a$ (~~173,000 SCCM~~), for all penetrations and valves subject to Type B and C tests when pressurized to P_s .

as specified in Specification 6.5.6, "Containment Leakage Rate Testing Program."

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION: With either (a) the measured overall integrated containment leakage rate exceeding the acceptance criteria specified in the Containment Leakage Rate Testing Program, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.50 L_a$, restore the overall integrated containment leakage rate to within the acceptance criteria specified in the Containment Leakage Rate Testing Program, and the combined leakage rate for all penetrations and valves subject to Types B and C tests to less than or equal to $0.50 L_a$ prior to increasing the Reactor Coolant System temperature above 200°F.

the Containment Leakage Rate Testing Program

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria, methods and provisions specified in 10 CFR Part 50, Appendix J:

a. Perform required visual examinations and Type A testing in accordance with the Containment Leakage Rate Testing Program.

schedule,

leakage rate testing, except for containment air lock testing,

3/4.6 CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- DELETE* →
- b. Type B and C tests shall be conducted with gas at P₁ (50 psig) at intervals of 24 months except for tests involving air locks.
 - c. Air locks shall be tested and demonstrated **OPERABLE** per Surveillance Requirement 4.6.1.3.
 - d. All Type B and C test leakage rates shall be calculated using observed data converted to absolute values.

- b. ~~4.6.1.2~~ Containment purge isolation valves shall be demonstrated **OPERABLE** any time upon entering **MODE 5** from power operation modes, unless the last surveillance test has been performed within the past six months or any time after being opened and prior to entering **MODE 4** from shutdown modes by verifying that when the measured leakage rate is added to the leakage rates determined pursuant to Technical Specification ~~4.6.1.2.b~~ for all other Type B or C penetrations, the combined leakage rate is less than or equal to ~~0.50 L (173,000 SCCM)~~. The leakage rate for the containment purge isolation valves shall also be compared to the previously measured leakage rate to detect excessive valve degradation.
- 4.6.1.2.a* →
- 0.60* → ~~0.50 L (173,000 SCCM)~~

- c. ~~4.6.1.3~~ The containment purge isolation valve seals shall be replaced with new seals at a frequency to ensure no individual seal remains in service greater than 2 consecutive fuel reload cycles.

DELETE →

* — Exemption to 10 CFR Part 50, Appendix J.

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

Containment Air Locks

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be **OPERABLE** with:

a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and

b. An overall air lock leakage rate of $\leq 0.05 \text{ L}$, ~~(17,300 SCCM)~~ at ~~P_a 50 psig~~, as specified in Specification 6.5.6, "Containment Leakage Rate Testing Program."

APPLICABILITY: **MODES 1, 2, 3, and 4.**

ACTION:

a. With an air lock inoperable, except as a result of an inoperable door gasket, restore the air lock to **OPERABLE** status within 24 hours or be in at least **HOT STANDBY** within the next 6 hours and in **COLD SHUTDOWN** within the following 30 hours.

b. With an air lock inoperable due to an inoperable door gasket:

1. Maintain the remaining door of the affected air lock closed and sealed, and

2. Restore the air lock to **OPERABLE** status within 7 days or be in at least **HOT STANDBY** within the next 6 hours and in **COLD SHUTDOWN** within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated **OPERABLE**:

a. After each opening, except when the air lock is being used for multiple entries, then at least once per 72 hours by verifying that the seal leakage is $\leq 0.0002 \text{ L}$ (69.2 SCOM) as determined by precision flow measurement when the volume between the door seals is pressurized to a constant pressure of 15 psig,

By performing containment air lock leakage rate testing in accordance with the Containment Leakage Rate Testing Program, and

* Exemption to 10 CFR Part 50, Appendix J.

DELETE

3/4.6 CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

b. At least once per 6 months by conducting an overall air lock leakage test at P₁ (50 psig) and by verifying that the overall air lock leakage rate is within its limit, and

b. ~~1.~~ At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

3/4.9 REFUELING OPERATIONS

3/4.9.8 SHUTDOWN COOLING AND COOLANT CIRCULATION

LIMITING CONDITION FOR OPERATION

3.9.8.1 At least one shutdown cooling loop shall be in operation.*

APPLICABILITY: **MODE 6** at all reactor water levels.

ACTION:

- a. With less than one shutdown cooling loop in operation*, suspend all operations involving an increase in the reactor decay heat load or a reduction in boron concentration of the Reactor Coolant System and, specifically, the charging pumps shall be de-energized and the charging flow paths shall be closed. Close all containment penetrations providing direct access from one containment atmosphere to the outside atmosphere within 4 hours. The shutdown cooling pumps may be de-energized during the time intervals required for local leak rate testing of containment penetration number 41 pursuant to the requirements of Specification 4.6.1.2.5^a and/or to permit maintenance on valves located in the common shutdown cooling suction line, provided (1) no operations are permitted which could cause dilution of the Reactor Coolant System boron concentration and, specifically, the charging pumps shall be de-energized and the charging flow paths shall be closed, (2) all **CORE ALTERATIONS** are suspended, (3) all containment penetrations providing direct access from the containment atmosphere to the outside atmosphere are maintained closed, and (4) the water level above the top of the irradiated fuel is greater than 23 feet.
- b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.8.1 A shutdown cooling loop shall be determined to be in operation and circulating reactor coolant at a flow rate of ≥ 1500 gpm at least once per 4 hours.

* The shutdown cooling loop may be removed from operation for up to 1 hour per 8 hour period during the performance of **CORE ALTERATIONS** in the vicinity of the reactor pressure vessel hot legs.

6.0 ADMINISTRATIVE CONTROLS

- h. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the site boundary, conforming to 10 CFR Part 50, Appendix I;
- i. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half lives > 8 days in gaseous effluents released from each unit to areas beyond the site boundary, conforming to 10 CFR Part 50, Appendix I; and
- j. Limitations on the annual dose or dose commitment to any member of the public due to releases to radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR Part 190.

6.5.6 Containment Leakage Rate Testing Program

A program shall be established to implement the leakage testing of the containment as required by 10 CFR 50.54(o) and 10 CFR Part 50, Appendix J, Option B, ~~as modified by approved exemptions~~. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995, ~~as modified by approved exceptions~~, *including errata.*

The peak calculated containment internal pressure for the design basis loss-of-coolant accident, P_a , is 49.4 psig. The containment design pressure is 50 psig.

The maximum allowable containment leakage rate, L_a , shall be 0.20 percent of containment air weight per day at P_a .

INSERT 6.5.6
~~Containment leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria is $\leq 0.75 L_a$ for Type A tests.~~

The provisions of Specification 4.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.

The provisions of Specification 4.0.3 are applicable to the Containment Leakage Rate Testing Program.

INSERT 6.5.6

Leakage rate acceptance criteria are:

- a. Containment leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first unit startup, following testing in accordance with this program, the leakage rate acceptance criteria are $\leq 0.60 L_a$ for Types B and C tests and $\leq 0.75 L_a$ for Type A tests.
- b. Air lock testing acceptance criteria are:
 1. Overall air lock leakage rate is $\leq 0.05 L_a$ when tested at $\geq P_a$.
 2. For each door, leakage rate is $\leq 0.0002 L_a$ when pressurized to ≥ 15 psig.

3/4.6 CONTAINMENT SYSTEMS

BASES

3/4.6.1 PRIMARY CONTAINMENT

3/4.6.1.1 CONTAINMENT INTEGRITY

In **MODES** 1, 2, 3, and 4, primary **CONTAINMENT INTEGRITY** ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the **SITE BOUNDARY** radiation doses to within the limits of 10 CFR Part 100 during accident conditions. In **MODES** 5 and 6, the probability and consequences of these events are reduced because of the Reactor Coolant System (RCS) pressure and temperature limitations of these modes, by preventing operations which could lead to a need for containment isolation, and by providing containment isolation through penetration closure.

3/4.6.1.2 Containment Leakage

Maintaining the containment **OPERABLE** requires compliance with the visual examinations and leakage rate test requirements of the Containment Leakage Rate Testing Program for Type A tests, and 10 CFR Part 50, Appendix J, Option A for Type B and C tests. As-kept leakage prior to the first startup after performing a required leakage test is required to be $\leq 0.75 L_a$ for overall Type A leakage. At all other times between required leakage rate tests, the acceptance criteria is based on an overall Type A leakage limit of $\leq 1.0 L_a$. At $\leq 1.0 L_a$, the offsite dose consequences are bounded by the assumptions of the safety analysis. The frequency of Type A testing is specified in the Containment Leakage Rate Testing Program.

The surveillance testing for measuring leakage rates are consistent with the requirements of 10 CFR Part 50, Appendix J, Option B for Type A tests. For Type B and C testing, the allowable leakage rate has been proportionately reduced, as recommended in Generic Letter 91-04, to account for an extended surveillance schedule of 24 months + 25% (per Specification 4.0.2). This is an exception from 10 CFR Part 50, Appendix J.

3/4.6.1.3 Containment Air Locks

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on **CONTAINMENT INTEGRITY** and containment leak rate. Surveillance testing of the air lock seals provides assurance that the overall air lock leakage will not become excessive due to seal damage during the intervals between air lock leakage tests.

The frequency of containment air lock leakage rate testing is specified in the Containment Leakage Rate Testing Program.

The overall air lock leakage rate is required to be $\leq 0.05 L_a$ (17,300 SCCM) at P_a , and $\leq 0.0002 L_a$ (69.2 SCCM) when the volume between the door seals is pressurized to 15 psig.

INSERT "3/4.6.1.2 - BASES"

INSERT "3/4.6.1.2 - BASES"

3/4.6.1.2 Containment Leakage

Maintaining the containment **OPERABLE** requires compliance with the visual examinations and leakage rate test requirements of the Containment Leakage Rate Testing Program. The surveillance testing for measuring leakage rates is consistent with the requirements of 10 CFR Part 50, Appendix J, Option B.

As-left leakage, prior to the first startup after performing a required leakage test, is required to be $\leq 0.60 L_a$ (207,600 SCCM) for combined Types B and C leakage, and $\leq 0.75 L_a$ (259,500 SCCM) for overall containment Type A leakage. At all other times between required leakage rate tests, the acceptance criteria is based on an overall containment leakage limit of $\leq 1.0 L_a$ (346,000 SCCM). At $\leq 1.0 L_a$, the offsite dose consequences are bounded by the assumptions of the safety analysis. Surveillance Requirement frequencies are as required by Appendix J.

These periodic testing requirements verify that the containment leakage rate does not exceed the leakage rate assumed in the safety analysis.

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary **CONTAINMENT INTEGRITY** shall be maintained.* †

APPLICABILITY: **MODES 1, 2, 3 and 4.**

ACTION: Without primary **CONTAINMENT INTEGRITY**, restore **CONTAINMENT INTEGRITY** within one hour or be in at least **HOT STANDBY** within the next 6 hours and in **COLD SHUTDOWN** within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary **CONTAINMENT INTEGRITY** shall be demonstrated:

- a. At least once per 31 days by verifying that all penetrations** not capable of being closed by **OPERABLE** containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except for valves that are open under administrative control as permitted by Specification 3.6.4.1.
- b. By verifying that each containment air lock is in compliance with the requirements of Specification 3.6.1.3.
- c. By verifying that the equipment hatch is closed and sealed, prior to entering **MODE 4** following a shutdown where the equipment hatch was opened, by conducting a Type B test per 10 CFR Part 50, Appendix J.

in accordance with the Containment Leakage Rate Testing Program.

* Hydrogen purge containment vent isolation valves shall be opened for containment pressure control, airborne radioactivity control, and surveillance testing purposes only.

† The shutdown cooling isolation valves may be opened when the RCS temperature is below 300°F to establish shutdown cooling flow.

** Except valves, blind flanges, and deactivated automatic valves which are located inside the containment and are locked, sealed, or otherwise secured in the closed position. These penetrations shall be verified closed during each **COLD SHUTDOWN** except that such verification need not be performed more often than once per 92 days.

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

Containment Leakage

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. A maximum allowable containment leakage rate, L_a , as specified in Specification 6.19, "Containment Leakage Rate Testing Program."

b. A combined leakage rate of $\leq 0.50 L_a$ (~~173,000 SCCM~~), for all penetrations and valves subject to Type B and C tests ~~when pressurized to P_s~~ .

as specified in Specification 6.5.6, "Containment Leakage Rate Testing Program."

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION: With either (a) the measured overall integrated containment leakage rate exceeding the acceptance criteria specified in the Containment Leakage Rate Testing Program, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.50 L_a$, restore the overall integrated containment leakage rate to within the acceptance criteria specified in the Containment Leakage Rate Testing Program, and the combined leakage rate for all penetrations and valves subject to Type B and C tests to less than or equal to $0.50 L_a$, prior to increasing the Reactor Coolant System temperature above 200°F.

the Containment Leakage Rate Testing Program

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates ~~shall be demonstrated at the following test schedule and~~ shall be determined in conformance with the criteria, methods, and provisions specified in ~~10 CFR Part 50, Appendix d:~~

a. Perform required visual examinations and ~~Type A testing~~ in accordance with the Containment Leakage Rate Testing Program.

leakage rate testing, except for containment air lock testing,

3/4.6 CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

DELETE

- b. Type B and C tests shall be conducted with gas at P₁ (50 psig) at intervals of 24 months except for tests involving air locks.
- c. Air locks shall be tested and demonstrated **OPERABLE** per Surveillance Requirement 4.6.1.3.
- d. All Type B and C test leakage rates shall be calculated using observed data converted to absolute values.

~~b.~~ ~~4.6.1.2.2~~ ~~0.60~~ ~~0.50 L (173,000 SCCM)~~ Containment purge isolation valves shall be demonstrated **OPERABLE** any time upon entering **MODE 5** from **POWER OPERATION MODES**, unless the last surveillance test has been performed within the past 6 months or any time after being opened and prior to entering **MODE 4** from shutdown modes by verifying that when the measured leakage rate is added to the leakage rates determined pursuant to Technical Specification ~~4.6.1.2.b~~ for all other Type B or C penetrations, the combined leakage rate is less than or equal to ~~0.50 L (173,000 SCCM)~~. The leakage rate for the containment purge isolation valves shall also be compared to the previously measured leakage rate to detect excessive valve degradation.

~~c.~~ ~~0.60~~ The containment purge isolation valve seals shall be replaced with new seals at a frequency to ensure no individual seal remains in service greater than 2 consecutive fuel reload cycles.

DELETE

* ~~Exemption to 10 CFR Part 50, Appendix J.~~

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

Containment Air Locks

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be **OPERABLE** with:

a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and

b. An overall air lock leakage rate of $\leq 0.05 \text{ L. (17,300 SCCM) at}$

~~P_a 50 psig.~~

as specified in Specification 6.5.6, "Containment Leakage Rate Testing Program."

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

a. With an air lock inoperable, except as a result of an inoperable door gasket, restore the air lock to **OPERABLE** status within 24 hours or be in at least **HOT STANDBY** within the next 6 hours and in **COLD SHUTDOWN** within the following 30 hours.

b. With an air lock inoperable due to an inoperable door gasket:

1. Maintain the remaining door of the affected air lock closed and sealed, and

2. Restore the air lock to **OPERABLE** status within 7 days or be in at least **HOT STANDBY** within the next 6 hours and in **COLD SHUTDOWN** within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated **OPERABLE**:

a. *After each opening, except when the air lock is being used for multiple entries, then at least once per 72 hours by verifying that the seal leakage is $\leq 0.0002 \text{ L. (69.2 SCCM)}$ as determined by precision flow measurement when the volume between the door seals is pressurized to a constant pressure of 15 psig.*

By performing containment air lock leakage rate testing in accordance with the Containment Leakage Rate Testing Program, and

* ~~Exemption to 10 CFR Part 50, Appendix J.~~

DELETE

3/4.6 CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- d. At least once per 6 months by conducting an overall air lock leakage test at P_a (50 psig) and by verifying that the overall air lock leakage rate is within its limit, and
- At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

3/4.9 REFUELING OPERATIONS

3/4.9.8 SHUTDOWN COOLING AND COOLANT CIRCULATION

LIMITING CONDITION FOR OPERATION

3.9.8.1 At least one shutdown cooling loop shall be in operation.*

APPLICABILITY: **MODE 6** at all reactor water levels.

ACTION:

- a. With less than one shutdown cooling loop in operation*, suspend all operations involving an increase in the reactor decay heat load or a reduction in boron concentration of the Reactor Coolant System and, specifically, the charging pumps shall be de-energized and the charging flow paths shall be closed. Close all containment penetrations providing direct access from the containment atmosphere to the outside atmosphere within 4 hours. The shutdown cooling pumps may be de-energized during the time intervals required for local leak rate testing of containment penetration number 41 pursuant to the requirements of Specification 4.6.1.2. ^a and/or to permit maintenance on valves located in the common shutdown cooling suction line, provided (1) no operations are permitted which could cause dilution of the Reactor Coolant System boron concentration and, specifically, the charging pumps shall be de-energized and the charging flow paths shall be closed, (2) all **CORE ALTERATIONS** are suspended, (3) all containment penetrations providing direct access from the containment atmosphere to the outside atmosphere are maintained closed, and (4) the water level above the top of the irradiated fuel is greater than 23 feet.
- b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.8.1 A shutdown cooling loop shall be determined to be in operation and circulating reactor coolant at a flow rate of ≥ 1500 gpm at least once per 4 hours.

*

The shutdown cooling loop may be removed from operation for up to 1 hour per 8 hour period during the performance of **CORE ALTERATIONS** in the vicinity of the reactor pressure vessel hot legs.

6.0 ADMINISTRATIVE CONTROLS

- h. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the site boundary, conforming to 10 CFR Part 50, Appendix I;
- i. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half lives > 8 days in gaseous effluents released from each unit to areas beyond the site boundary, conforming to 10 CFR Part 50, Appendix I; and
- j. Limitations on the annual dose or dose commitment to any member of the public due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR Part 190.

6.5.6 Containment Leakage Rate Testing Program

A program shall be established to implement the leakage testing of the containment as required by 10 CFR 50.54(o) and 10 CFR Part 50, Appendix J, Option B. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995.

The peak calculated containment internal pressure for the design basis loss-of-coolant accident, P_a , is 49.4 psig. The containment design pressure is 50 psig.

The maximum allowable containment leakage rate, L_a , shall be 0.20 percent of containment air weight per day at P_a .

Containment leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria is $\leq 0.75 L_a$ for Type A tests.

The provisions of Specification 4.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.

The provisions of Specification 4.0.3 are applicable to the Containment Leakage Rate Testing Program.

INSERT 6.5.6

Leakage rate acceptance criteria are:

- a. Containment leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first unit startup, following testing in accordance with this program, the leakage rate acceptance criteria are $\leq 0.60 L_a$ for Types B and C tests and $\leq 0.75 L_a$ for Type A tests.
- b. Air lock testing acceptance criteria are:
 1. Overall air lock leakage rate is $\leq 0.05 L_a$ when tested at $\geq P_a$.
 2. For each door, leakage rate is $\leq 0.0002 L_a$ when pressurized to ≥ 15 psig.

3/4.6 CONTAINMENT SYSTEMS

BASES

3/4.6.1 PRIMARY CONTAINMENT

3/4.6.1.1 CONTAINMENT INTEGRITY

In **MODES** 1, 2, 3, and 4, primary **CONTAINMENT INTEGRITY** ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the accident analyses. This restriction, in conjunction with the leakage rate limitation, will limit the **SITE BOUNDARY** radiation doses to within the limits of 10 CFR Part 100 during accident conditions. In **MODES** 5 and 6, the probability and consequences of these events are reduced because of the Reactor Coolant System (RCS) pressure and temperature limitations of these modes, by preventing operations which could lead to a need for containment isolation, and by providing containment isolation through penetration closure.

3/4.6.1.2 Containment Leakage

Maintaining the containment **OPERABLE** requires compliance with the visual examinations and leakage rate test requirements of the Containment Leakage Rate Testing Program for Type A tests, and 10 CFR Part 50, Appendix J, Option A for Type B and C tests. As-kept leakage prior to the first startup after performing a required leakage test is required to be $\leq 0.75 L_a$ for overall Type A leakage. At all other times between required leakage rate tests, the acceptance criteria is based on an overall Type A leakage limit of $\leq 1.0 L_a$. At $\leq 1.0 L_a$, the offsite dose consequences are bounded by the assumptions of the safety analysis. The frequency of Type A testing is specified in the Containment Leakage Rate Testing Program.

The surveillance testing for measuring leakage rates are consistent with the requirements of 10 CFR Part 50, Appendix J, Option B for Type A tests. For Type B and C testing, the allowable leakage rate has been proportionately reduced, as recommended in Generic Letter 91-04, to account for an extended surveillance schedule of 24 months + 25% (per Specification 4.0.2). This is an exception from 10 CFR Part 50, Appendix J.

3/4.6.1.3 Containment Air Locks

The limitations on closure and leak rate for the containment air locks are required to meet the restrictions on **CONTAINMENT INTEGRITY** and containment leak rate. Surveillance testing of the air lock seals provides assurance that the overall air lock leakage will not become excessive due to seal damage during the intervals between air lock leakage tests.

The frequency of containment air lock leakage rate testing is specified in the Containment Leakage Rate Testing Program.

The overall air lock leakage rate is required to be $\leq 0.05 L_a$ (17,300 SCCM) at P_a , and $\leq 0.0002 L_a$ (69.2 SCCM) when the volume between the door seals is pressurized to 15 psig.

INSERT "3/4.6.1.2 - BASES"

3/4.6.1.2 Containment Leakage

Maintaining the containment **OPERABLE** requires compliance with the visual examinations and leakage rate test requirements of the Containment Leakage Rate Testing Program. The surveillance testing for measuring leakage rates is consistent with the requirements of 10 CFR Part 50, Appendix J, Option B.

As-left leakage, prior to the first startup after performing a required leakage test, is required to be $\leq 0.60 L_a$ (207,600 SCCM) for combined Types B and C leakage, and $\leq 0.75 L_a$ (259,500 SCCM) for overall containment Type A leakage. At all other times between required leakage rate tests, the acceptance criteria is based on an overall containment leakage limit of $\leq 1.0 L_a$ (346,000 SCCM). At $\leq 1.0 L_a$, the offsite dose consequences are bounded by the assumptions of the safety analysis. Surveillance Requirement frequencies are as required by Appendix J.

These periodic testing requirements verify that the containment leakage rate does not exceed the leakage rate assumed in the safety analysis.