

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 1 3 8 1 8										PAGE (3) 1 OF 0 1															
TITLE (4) ESF Actuation (RWCU Isolation on High System Flow).																																			
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																	
MONTH		DAY		YEAR		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH		DAY		YEAR		FACILITY NAMES																	
																		DOCKET NUMBER(S)																	
																		0 5 0 0 0 1 3 8 1 8																	
0 8		0 7		8 5		8 5		0 2		4		0 0		0 8		3 0		8 5		0 5 0 0 0 1 3 8 1 8															
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																																	
2		20.402(b)										20.406(a)										<input checked="" type="checkbox"/> 50.73(a)(2)(iv)		73.71(b)											
POWER LEVEL (10)		01.013										20.406(a)(1)(i)										50.36(a)(1)										<input type="checkbox"/> 50.73(a)(2)(iv)		73.71(b)	
		20.406(a)(1)(ii)										50.36(a)(2)										<input type="checkbox"/> 50.73(a)(2)(iv)		OTHER (Specify in Abstract below and in Text, NRC Form 306A)											
		20.406(a)(1)(iii)										50.73(a)(2)(i)										<input type="checkbox"/> 50.73(a)(2)(v)(A)													
		20.406(a)(1)(iv)										50.73(a)(2)(ii)										<input type="checkbox"/> 50.73(a)(2)(v)(B)													
		20.406(a)(1)(v)										50.73(a)(2)(iii)										<input type="checkbox"/> 50.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																																			
NAME L.A. Kuczynski - Nuclear Plant Specialist, Level III																TELEPHONE NUMBER 7 1 1 7 5 4 1 2 - 1 3 1 7 1 5 1 9																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																			
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC		CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC																	
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR													
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)																<input checked="" type="checkbox"/> NO																			

ABSTRACT (Limit to 1400 words. Use approximately three single-space typewritten lines) (16)

On August 7, 1985, the reactor water cleanup (RWCU) containment inboard isolation valve closed on a high system flow signal generated while RWCU filter/demineralizer (f/d) 'B' was being placed in service. (This valve is part of the primary containment isolation system, which is an Engineered Safety Feature.) RWCU system operation was restored to normal with f/d 'B' in service. An engineering evaluation of the transient has determined the most probable cause of the high flow was the fast opening of the f/d inlet valve. The next time the f/d is removed from service, the inlet valve's stroke time will be checked and adjusted as necessary. Additionally, the RWCU operating procedure will be revised to remove the precoat tank from the recirculation path during the time that samples of the resin slurry are being taken and analyzed. This will minimize air entrainment via the precoat tank while RWCU is in the precoat cycle, and prevent an air buildup in the f/d which could then be compressed when the f/d is placed in service and yield a high system flow signal.

8509100145 850830
PDR ADOCK 05000388
S PDRIE22
11



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

August 30, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 85-024-00
ER 100450 FILE 841-23
PLAS- 109

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 85-024-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that the Unit experienced the unanticipated actuation of an Engineered Safety Feature when the Reactor Water Cleanup System containment inboard isolation valve closed on a high system flow signal.

T.M. Crimmins, Jr.
Superintendent of Plant-Susquehanna

LAK/pjg

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Mr. R.H. Jacobs
Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 52
Shickshinny, PA 18655

1E22
1/1