

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)

Zion Unit 2

DOCKET NUMBER (2)

050003041 OF 02

PAGE (3)

TITLE (4)

Operation with an Inoperable Wide Range Reactor Coolant Pressure Channel

| EVENT DATE (5) |     |      | LER NUMBER (6) |                   |                 | REPORT DATE (7) |     |      | OTHER FACILITIES INVOLVED (8) |                  |  |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|------------------|--|
| MONTH          | DAY | YEAR | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH           | DAY | YEAR | FACILITY NAME                 | DOCKET NUMBER(S) |  |
| 06             | 02  | 83   | 84             | 006               |                 | 04              | 04  | 84   |                               | 050003           |  |

| OPERATING MODE (9) | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50. (Check one or more of the following) (11) |              |  |             |  |                 |  |  |  |  |  |
|--------------------|---|--------------|--|-------------|--|-----------------|--|--|--|--|--|
| 1                  |   |              |  |             |  |                 |  |  |  |  |  |
| POWER LEVEL (10)   | 01919   | 20.4022M     |  | 20.4084M    |  | 60.7361C20M     |  | 73.718M  |  |  |  |
|                    |   | 20.4084M(1)M |  | 60.3861C1   |  | 60.7361C21M     |  | 73.716M  |  |  |  |
|                    |   | 20.4084M(1)M |  | X 60.3861C2 |  | 60.7361C21M     |  | OTHER (Specify in Abstract below and in Text, NRC Form 300A) |  |  |  |
|                    |   | 20.4084M(1)M |  | 60.7361C21M |  | 60.7361C21M(1A) |  |  |  |  |  |
|                    |   | 20.4084M(1)M |  | 60.7361C21M |  | 60.7361C21M(1B) |  |  |  |  |  |
|                    |   | 20.4084M(1)M |  | 60.7361C21M |  | 60.7361C21M     |  |  |  |  |  |

| LICENSEE CONTACT FOR THIS LER (12) |  |  |  |  |  |  |  |                  |  |  |
|------------------------------------|--|--|--|--|--|--|--|------------------|--|--|
| NAME                               |  |  |  |  |  |  |  | TELEPHONE NUMBER |  |  |
| X. Polanski                        |  |  |  |  |  |  |  | 3112 71461-12084 |  |  |

| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) |        |           |              |                   |       |        |           |              |                   |  |
|--|--------|-----------|--------------|-------------------|-------|--------|-----------|--------------|-------------------|--|
| CAUSE  | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC |  |
| X  |        |           |              | N                 |       |        |           |              |                   |  |
|  |        |           |              |                   |       |        |           |              |                   |  |
|  |        |           |              |                   |       |        |           |              |                   |  |

| SUPPLEMENTAL REPORT EXPECTED (14)               |  |  |  | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|---|--|--|--|-------------------------------|-------|-----|------|
| YES (If yes, complete EXPECTED SUBMISSION DATE) |  |  |  | X NO                          |       |     |      |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A routine operating surveillance channel check showed that the two wide range reactor coolant system pressure channels differed by more than the allowed tolerance. This condition went uncorrected until the next monthly surveillance was performed, at which time, the faulty channel was repaired. That channel was inoperable for more than 7 days, violating Tech. Spec. 3.8.9.a. The cause of the violation was failure of shift personnel to recognize the Tech. Spec. significance of the instrument at the time the problem was first discovered. To prevent recurrence, additional notes and precautions regarding Technical Specifications were added to the test procedure.

8509090304 840404  
PDR ADOCK 05000304  
S PDRIE 22  
11

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104  
EXPIRES: 8/31/88

|                                      |  |                |                   |                 |          |        |
|--------------------------------------|--|----------------|-------------------|-----------------|----------|--------|
| FACILITY NAME (1)<br><br>Zion Unit 2 | DOCKET NUMBER (2)<br><br>0 5 0 0 0 3 0 4 8 7 | LER NUMBER (6) |                   |                 | PAGE (3) |        |
|                                      |  | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER |          |        |
|                                      |  | 0 0 6          | 0 0 6             | 0 0 0           | 0 2      | OF 0 2 |

TEXT (If more space is required, use additional NRC Form 388A (1/77))

On June 2, 1983, during normal operation at full power, the monthly "Accident Monitoring Instrumentation Channel Check Test, PT-2G, was performed. Wide range reactor coolant system pressure channels 403 and 405 were found to differ by 135 p.s.i., exceeding the allowed tolerance of 45 p.s.i. A work request was written, but the channel was not repaired. On June 27, PT-2G was performed again, and wide range R.C.S. pressure was again found out of tolerance. A second work request was written, and on the next day the faulty channel, 405, was repaired. On July 20, when PT-2G was again performed, the wide range R.C.S. pressure channels were within tolerance.

Tech. Spec. 3.8.9.a. requires two operable channels in modes 1, 2, and 3, and it allows one channel to be inoperable for seven days. Channel 405 was inoperable for 26 days, yet the plant remained in Mode 1. The cause of this Tech. Spec. L.C.O. violation was failure of shift personnel to recognize the Tech. Spec. requirements for accident monitoring instrumentation.

Corrective actions included the addition of notes and precautions to PT-2G. They caution shift personnel to review Tech. Spec. 3.8.9.a. and b. in the event of any channels being out-of-service, inoperable, or out of tolerance. The I.M. Department reviewed all outstanding work requests to insure no other work was pending on post accident monitoring instrumentation. All PT's with discrepancies noted will be routed to the Operating Engineer for review for the near term. The Tech. Spec. requirements for post accident monitoring instrumentation will be reviewed during license renewal training.



Commonwealth Edison

## DEVIATION REPORT

DVR NO. 22 - 2 - 84 - 14  
STA UNIT YEAR NO.

## PART 1 TITLE OF DEVIATION

Degraded Post Accident Inst.

OCCURRED 6-2-83

SYSTEM AFFECTED

RC

PLANT STATUS AT TIME OF EVENT

MODE Oper. POWER(%) 99%

DATE TIME

26197

TESTING

WORK REQUEST NO.

☐ YES ☒ NO

## DESCRIPTION OF EVENT

In reviewing outage WR's, an outstanding WR on U-2 wide range reactor coolant pressure was found to exist. In reviewing PT-2G for post accident monitoring instrumentation, it was found that the W.R. pressures were out of tolerance on 6-2-83 and on 6-27-83. They were in tolerance on 7-20-83. The 7 day clock on inoperable post accident monitoring equipment appears to have been exceeded.

POTENTIALLY SIGNIFICANT EVENT PER NSD DIRECTIVE A-07

☐ YES ☒ NO10CFR50.72 NRC RED PHONE  
NOTIFICATION MADE☐ 1 HOUR  
☒ 4 HOUR

TIME

☒ NOJ. Gilmore  
RESPONSIBLE SUPERVISOR3-9-84  
DATE

## PART 2 OPERATING ENGINEER'S COMMENTS

Additional notes & cautions have been added to PT-2G.

☐ NON REPORTABLE EVENT☒ 30 DAY REPORTABLE/10CFR 50.36☐ 5 DAY REPORT PER 10CFR21☐ ANNUAL/SPECIAL REPORT REQUIREDNOTIFICATION F. Dunaway 3-9-84 10:00 AM  
REGION III DATE TIME

NSD

DATE

TIME

☐ CECO CORPORATE NOTIFICATION MADE  
IF ABOVE NOTIFICATION IS PER 10CFR21TELECOPY D. P. Galle 3-13-84  
CECO CORPORATE OFFICER DATE TIME

A.I.R. #

L.E.R. # 50-304/84-006/00

PRELIMINARY REPORT  
COMPLETED AND REVIEWED

J. Gilmore

3-9-84

OPERATING ENGINEER

DATE

INVESTIGATION REPORT & RESOLUTION  
ACCEPTED BY STATION REVIEW

C. Schmidt ARG 4/3/84

RESOLUTION APPROVED AND  
AUTHORIZED FOR DISTRIBUTION

STATION SUPERINTENDENT

4-3-84  
DATE

(Form 15-52-1)

1-84



Commonwealth Edison  
Zion Generating Station  
Shiloh Blvd. & Lake Michigan  
Zion, Illinois 60099  
Telephone 312/746-2084

PC  
5-11-84

0001147 A12

April 4, 1984

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

References: 10CFR36  
NUREG 1022

Dear Sir:

The enclosed Licensee Event Report from Zion Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.36(c)(2) which requires a 30 day written report when there has been a failure to meet a Technical Specification Limiting Condition for Operation.

This report is number 84-006-00, Docket No. 50-304/DPR-48.

Very truly yours,

*K. L. Graesser*

K. L. Graesser  
Superintendent  
Zion Generating Station

Enclosure: Licensee Event Report No. 84-006-00

Attachment

cc: J. G. Keppler, NRC Region III Administrator  
M. Holzmer, NRC Resident Inspector  
INPO Record Center  
CECO Distribution List