



50-458

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 24, 1997

Mr. John R. McGaha, Jr.
Vice President - Operations
Entergy Operations, Inc.
River Bend Station
P. O. Box 220
St. Francisville, LA 70775

SUBJECT: RIVER BEND STATION, UNIT 1 - NOTIFICATION OF CONDUCT OF A FIRE
PROTECTION FUNCTIONAL INSPECTION (FPFI)

Dear Mr. McGaha:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) staff will conduct a team inspection in the fire protection subject area at River Bend Station the weeks of April 21 and May 5, 1997. The Fire Protection Functional Inspection (FPFI) team will be lead by Mr. Patrick Madden, a senior fire protection engineer in the Office of Nuclear Reactor Regulation (NRR). The FPFI team will be composed of personnel from NRR, NRC Region IV, NRC Region II, and Brookhaven National Laboratory.

The staff described the FPFI program in detail in SECY-96-267, "Fire Protection Functional Inspection Program," dated December 24, 1996. A copy of this paper is included for your information as Enclosure 1. The FPFI procedure itself will be publicly available no later than mid-April 1997.

The scope of this inspection is to review the implementation of your fire protection program. The FPFI team will evaluate:

- Fire Protection Program Administration
- Fire Protection Systems and Features
- Post-fire Safe Shutdown Capability and Implementation
- Fire Protection Configuration Management Program and Implementation
- Potential Fire Related Vulnerabilities
- Event Based Fires
- Fire Induced Plant Transients
- Seismic/fire Interaction

In order for the staff to prepare for and perform this comprehensive and complex inspection, members of the inspection team will visit River Bend Station on March 19 and 20, 1997. The purpose of this trip is to familiarize the team members with the River Bend Station, its fire protection program and its basis, and to obtain program related information and documentation. The types of documents these team members will be interested in reviewing, and possibly obtaining, are listed in Enclosure 2.

During the onsite inspection periods, we request that you ensure that copies of other analyses, evaluations or documentation regarding the implementation and maintenance of the River Bend station fire protection program, including post-fire safe shutdown capability, are readily accessible to the team for

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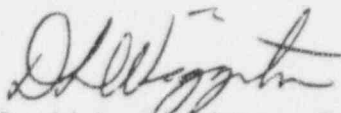
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their review. Of specific interest are those documents which establish that your fire protection program satisfies NRC regulatory requirements and conforms to applicable NRC fire protection guidance. Also, appropriate personnel, knowledgeable with respect to those plant systems required to achieve and maintain safe shutdown conditions from inside and outside the control room (including the electrical aspects of the post-fire safe shutdown analyses), reactor plant fire protection systems, and the River Bend fire protection program and its implementation should be available at the site during the inspection.

Approximately 2 weeks prior to the first week of the onsite inspection period, the inspection team plans to hold a conference call with cognizant members of your staff to discuss administrative inspection details such as: office space size and location; specific documents requested to be made available to the team in their office spaces; arrangements for reactor site access (including radiation protection training, security, safety and fitness for duty requirements [to be discussed only if not resolved during the information gathering visit]); and the availability of knowledgeable plant engineering and licensing organization personnel to serve as points of contact and/or escorts during the inspection.

Your cooperation and support during this inspection will be appreciated. If you have questions concerning this inspection or the inspection team's information or logistical needs, please contact Patrick M. Madden, the FPFI team leader, at 301-415-2854. If you have questions regarding the FPFI program itself, please contact Leon Whitney, the FPFI Program Manager, at 301-415-3081.

Sincerely,



David L. Wigginton, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-458

Enclosures: 1. SECY-96-267
2. Typical Fire Protection Program
Supporting Documentation

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Sincerely,

ORIGINAL SIGNED BY:

David L. Wigginton, Senior Project Manager
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-458

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2. Typical Fire Protection Program
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DISTRIBUTION: Docket File PUBLIC PD4-1 r/f SPLB r/f
CHawes JRoe DWigginton AHowell, RIV ACRS
OGC (15B18) EAdensam (EGA1) WBeckner LWhitney PQuals
WMiller JHyslop Plant r/f

Document Name: RBNDFPFI.LTR *See previous concurrence

OFC	FPFI TM LDR*	SC/FPES/SPLB*	PM/PD4-1	(A)LA/PD4-1
NAME	PMadden	SWest	DWigginton/vw	CHawes CMN
DATE	02/20/97	02/20/97	2/24/97	2/21/97
COPY	YES/NO	YES/NO	YES/NO	YES/NO

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POLICY ISSUE

(NEGATIVE CONSENT)

December 24, 1996

SECY-96-267

FOR: The Commissioners
FROM: James M. Taylor
Executive Director for Operations
SUBJECT: FIRE PROTECTION FUNCTIONAL INSPECTION PROGRAM

PURPOSE:

To inform the Commission of the objectives, scope, and status of the new fire protection functional inspection program and of the staff's plan to implement the program.

BACKGROUND:

In a memorandum of August 25, 1992, the staff of the U.S. Nuclear Regulatory Commission (NRC) submitted to the Commission its action plan for resolving the Thermo-Lag fire barriers issues. The staff stated that it would develop and implement a program to inspect the Thermo-Lag corrective actions at each plant. At that time, the staff believed that the licensees would simply replace or upgrade their existing Thermo-Lag fire barriers. However, since that time, the licensees have proposed a much broader range of corrective action options. For example, many licensees have initiated fire barrier reduction programs. The objective of these programs, which are based largely on reassessments and subsequent revisions of the plant post-fire safe shutdown analysis, is to eliminate as much as possible the need for fire barriers. Typical outcomes of barrier reduction programs include redefined fire area boundaries, new or relocated safe shutdown components, and new operator actions and procedures. Many licensees are also performing engineering

NOTE: TO BE MADE PUBLICLY AVAILABLE WHEN
THE FINAL SRM IS MADE AVAILABLE

CONTACTS:

Leon Whitney, NRR
301-415-3081

Steven West, NRR
301-415-1220

ENCLOSURE 1

evaluations to justify either eliminating certain Thermo-Lag barriers or keeping them as they are (i.e., without upgrades). In some cases, the licensees have used such evaluations to justify exemptions from the NRC fire protection regulations, and we anticipate the trend to continue in the future.

In the memorandum of August 25, 1992, the staff also informed the Commission that it would reassess the NRC reactor fire protection program to (1) determine if the program had appropriately addressed the safety issues, (2) determine if licensees are maintaining compliance with the NRC fire protection requirements, (3) identify the strengths and weaknesses of the program, and (4) make recommendations for improvement. The staff issued its "Report on the Reassessment of the NRC Fire Protection Program" on February 27, 1993. That report recommended, in part, that the staff (1) develop a coordinated approach for the fire protection and systems inspections and (2) reevaluate the scope of the fire protection inspection program. In SECY-93-143, "NRC Staff Actions To Address the Recommendations in the Report on the Reassessment of the NRC Fire Protection Program" dated May 21, 1993, the staff informed the Commission that it would implement these reassessment recommendations as part of the Fire Protection Task Action Plan. To do so, the staff considered fire events, licensee reports of deficiencies in the fire protection program, previous NRC inspection findings, the scope and adequacy of the existing NRC fire protection inspection program, and the need to inspect other plant fire protection features in response to ongoing NRC programs (e.g., self-induced station blackout, fire barrier penetration seals, turbine building assessments, and individual plant evaluations of external events (IPEEs)).

On the basis of the wide range of Thermo-Lag corrective actions proposed by the licensees, the staff concluded that an inspection of broader scope than that proposed in the Thermo-Lag Action Plan was needed. In addition, in view of the preliminary results of its work under the reassessment recommendation, the staff concluded that additional fire protection inspection effort appeared to be warranted. In SECY-95-034, "Status of Recommendations Resulting from the Reassessment of the NRC Fire Protection Program," dated February 13, 1995, the staff informed the Commission that it was considering initiating a fire protection functional inspection (FPFI) program, which would cover all aspects of nuclear power plant fire safety (including Thermo-Lag fire barriers) and provide for more efficient, comprehensive and effective inspections. Revision and/or cancellation of some of the existing fire protection inspection procedures will be considered as part of the FPFI program.

In a memorandum to the Commission of September 20, 1995, the staff documented its conclusion that an inspection of broader scope than that originally specified in the Thermo-Lag Action Plan was needed. The staff also informed the Commission that instead of the stand-alone Thermo-Lag fire barrier inspection program that it had proposed, it would develop and implement the FPFI program it had outlined in SECY-95-034. On February 8, 1996, the staff briefed the Chairman on its plans for the future direction of the NRC reactor fire protection program including the FPFI program. Later, in a memorandum to the Commission of April 3, 1996, the staff documented the framework for future direction of the NRC fire protection program with emphasis on the FPFI

program, a plan for developing and implementing this program, and a plan for centralized management, by the Office of Nuclear Reactor Regulation (NRR), of the FPGI program and all other reactor fire protection work.

The staff presented its plans for the FPGI program to the Advisory Committee on Reactor Safeguards in March 1996. The staff also presented its plans to the nuclear industry at the Regulatory Information Conference of May 1996 and at the Nuclear Energy Institute fire protection forum of September 1996.

DISCUSSION:

Objectives of the FPGI Program

The FPGI program is a new headquarters-based inspection program. The program satisfies a number of objectives. The program satisfies staff plans to inspect Thermo-Lag fire barrier corrective actions. The program also responds to the reassessment recommendation that the staff reevaluate the scope of the reactor fire protection inspection program and develop a coordinated approach for the fire protection and post-fire safe shutdown systems inspections. In this regard, the FPGI program will provide a strong, broad-based, and coherent inspection program that is commensurate with the safety significance of the subject and that will help ensure licensee compliance with NRC fire protection regulations and commitments. Benefits of the FPGI program include focusing NRC fire protection and support staff resources on the fire protection issues of most importance, (such as licensee control of the fire protection design and licensing bases), providing clear guidance to the staff and the nuclear industry regarding NRC oversight of licensee reactor fire protection programs, and improving the consistency of internal NRC oversight of the program. The program will also provide an immediate safety benefit arising from renewed industry attention to nuclear power plant fire safety.

Scope of FPGI Program

The FPGIs will be announced inspections and will cover all aspects of plant fire safety. The staff will use risk insights to help focus the FPGIs on those areas most important to safety. The principal focus of the inspections will be on the plant fire protection and post-fire safe shutdown design and licensing bases and those fire protection program elements that are covered by existing NRC regulations and guidelines. These include, for example, safe shutdown performance objectives, safe shutdown systems and equipment, fire protection systems and barriers, emergency lighting, reactor coolant pump oil collection systems, quality control and quality assurance, configuration control including change control process, administrative controls and procedures, and training. This aspect of the FPGI program will satisfy the program objective of ensuring continued licensee compliance with NRC fire protection regulations and commitments. In addition, the pilot inspections will include a review of fire safety considerations that are not expressly addressed by the fire protection regulation, but by other regulatory programs. This includes, principally, Generic Letter 88-20, Supplement 4, "Individual Plant Examinations of External Events (IPEEE) for Severe Accident Vulnerabilities, 10 CFR 50.54(f)," June 28, 1991. Such inspection areas

include, for example, event initiated fires, fire induced reactor transients, and potential seismic fire interactions. This feature of the FPGI program will provide useful information regarding broader aspects of nuclear power plant fire safety. The staff will use this information to identify the strengths and weaknesses of the overall NRC reactor fire protection program and to develop and support recommendations for program improvement, where appropriate.

The FPGI program consists of a pilot program followed by a permanent program. The pilot program consists of four pilot inspections (one per region) conducted in series over a 1-year period. The permanent program, as currently envisioned, would consist of four to eight NRC team inspections (one to two per region) per year. Licensee self-assessments could also be an important element of the permanent FPGI program. The staff will consider the role of self-assessments after it completes the pilot program.

A typical FPGI team will consist of a qualified team leader and four qualified inspectors. The team leader will be a senior fire protection engineer or equivalent. The team will consist of a fire protection engineer, an electrical engineer, a plant systems engineer, and a regional inspector. A probabilistic risk assessment (PRA) specialist will help with inspection preparation by developing plant-specific risk-informed information for the inspection plan. The use of region-based inspectors will help maintain region office involvement in the program. The regional inspectors should have standard regional inspector qualifications. Experience in fire protection and plant systems inspections would be an advantage, but not required. The inspection activities that will be assigned to the regional inspectors will depend on their individual experience and qualifications. Specific inspection assignments will be made by the team leader as part of inspection plan development. Examples include surveillance, testing, and repair activities and procedures; administrative controls; and quality assurance and quality controls.

For planning purposes, each FPGI will take 7 to 10 weeks (2 to 3 weeks to prepare, 2 weeks on site with a 1 week break between 1-week site visits, and 2 to 4 weeks to write the report). The first week of the onsite inspection will consist of a broad-based inspection of the plant's overall fire protection and post-fire safe shutdown program. During the second week, the team will inspect areas of emphasis based on the results of the first week of the inspection. For example, if during the first week the team finds as part of the basic inspection program that the licensee has a weak configuration control program, the team could inspect this program in depth during the second week. The inspection team's findings will be sufficiently developed to support enforcement actions, as appropriate. Enforcement actions will be processed by the regional offices with the assistance of NRR and the Office of Enforcement (OE) in accordance with NUREG-1600, "General Statement of Policy and Procedures for NRC Enforcement Actions," and the "NRC Enforcement Manual." (Although the inspections will include review of fire safety considerations that are not expressly addressed by the fire protection regulation, as discussed above, information obtained from this aspect of the inspections should not lead to enforcement actions.) With respect to technical issues,

decisions regarding responsibility for follow up activities will be made on a case-by-case basis after the FPGI. The staff expects that in most cases, the regional offices will perform the follow up activities. However, in some rare cases it may be more appropriate or more efficient for NRR to follow up.

The FPGI procedure will be "modular" in that sections of the procedure that address discrete inspection topics could be conducted by individual inspectors independent of a full-scale FPGI. For example, under the Thermo-Lag Action Plan, the staff prepared a draft Thermo-Lag fire barrier inspection procedure. This procedure will be converted into a fire barrier inspection procedure and integrated into the FPGI procedure. The staff could use this element of the FPGI program to inspect Thermo-Lag fire barriers independent of an FPGI, where appropriate. For example, it could be used at a plant that upgraded its Thermo-Lag fire barriers, but did not use the more complicated corrective action options discussed previously.

In general, the conduct of FPGIs will parallel that of other NRC team inspections. Before each inspection, the NRC team leader will assemble the inspection team, coordinate inspection preparation activities with the licensee and the individual team members, and prepare an inspection plan. During the inspection, the team leader will manage the implementation of the inspection plan, manage the inspection team, and interface with the licensee. After the inspection, the team leader will integrate the team member inputs into a comprehensive NRC inspection report. In addition, after each pilot inspection, the inspection team and appropriate NRC staff and management will critique the inspection and assess lessons learned. The staff will apply the results of the critiques and the lessons learned in the development of the inspection plan for the following pilot inspection.

After the four pilot inspections, the staff will reassess the lessons learned and modify the draft FPGI procedure and guidance to reflect the lessons learned. The revisions will be the final draft of the FPGI procedure and guidance. The staff estimates that it could complete this effort no later than 4 months after it completes the final pilot inspection. After the staff prepares the final draft FPGI procedure and guidance, it will conduct a public workshop regarding the FPGI program. During the workshop, the staff will discuss the FPGI program, present the results of the pilot program, and seek public and industry input. At this time, the staff will also consider the value of retaining those inspection elements that cover fire safety considerations that are not expressly addressed by the fire protection regulation. The staff estimates that it will conduct the workshop about one month after it completes the final draft of the FPGI procedure and guidance.

After the public workshop, the staff will finalize the FPGI procedure and guidance. At that time, the staff will also consider the need for training for additional FPGI inspectors. Depending on the extent of the comments and input received during the workshop, the staff estimates that it will issue the final FPGI procedure and guidance 2 to 3 months after the workshop. In addition, as part of its overall efforts to improve the effectiveness of NRC reactor fire protection inspections, the staff will reassess the existing core

fire protection inspection procedure (IP 64704, "Fire Protection Program") and develop recommendations regarding its future use or revision.

Current Status of FPFI Program Development

The Fire Protection Engineering Section of NRR will develop and implement the FPFI program. The staff is developing the FPFI procedure with technical assistance from Brookhaven National Laboratory and Sciencetech, Incorporated.

The staff has prepared a detailed outline of the FPFI program and procedures which have, in summary, the following major features:

- use of risk insights (PRA and IPEEE, when available)
- first week (core or basic inspection elements)
 - fire protection design and licensing bases
 - fire protection program
 - post-fire safe shutdown capability
- second week (in-depth inspection elements)
 - fire protection features, organization, controls, and practices
 - post-fire safe shutdown implementation
 - configuration control and management
 - event initiated fires
 - fire induced reactor transients
 - seismic fire interaction
- enforcement actions, as appropriate
- inspection follow up activities, as needed

Using this outline, the staff has prepared an initial draft of the FPFI procedure and guidance. NRR and the regions have selected four pilot plants (one per region) using criteria such as: the magnitude and character of licensee Thermo-Lag corrective action programs, whether licensees have been proactive in implementing their fire protection programs, reactor plant fire protection enforcement history, and the strength of licensee configuration management programs. The NRR staff is working with regional offices to schedule the four pilot FPFI inspections. As discussed below, the staff plans to begin the pilot inspections during the first quarter of calendar-year 1997.

Integration With Other NRC Programs

In SECY-96-134, "Options for Pursuing Regulatory Improvement in Fire Protection Regulations for Nuclear Power Plants," dated June 21, 1996, the staff asked the Commission to approve an option for improving the fire protection regulations. In a staff requirements memorandum of October 2, 1996, the Commission approved the staff recommendation to revise the current regulation. The Commission also stated that the staff should

consider such issues as inspection and enforcement in a manner consistent with that stated in the Commission's preliminary views on Direction Setting Issue 12 regarding risk-informed, performance-based regulation. The staff will follow this direction. In addition, the staff will coordinate development of the FPGI program with the fire protection rulemaking effort. The staff will ensure that the FPGI procedure is appropriate for the current fire protection regulation and will update the procedure, as appropriate, when it revises the regulation.

Impact on Licensees

Licensee support for an FPGI will be equivalent to that needed for other comprehensive team inspections. During inspection preparation, the licensee site and engineering organizations will provide such information as results of licensee fire protection audits, reviews, and self-assessments; fire hazards analyses; post-fire safe shutdown analyses; design change control packages; procedures; and drawings. While the team is on-site, experienced and knowledgeable licensee personnel will be required to support the team's inspection activities. These personnel will coordinate answers to the inspectors' questions, and provide design drawings, plant procedures, and other documents as needed. The staff also expects that site engineering and licensing managers will participate in entrance and exit meetings.

NRC Staff Resource Implications

The staff will use headquarters and regional staff and technical assistance contractors to conduct the FPGIs. For planning purposes, the staff assumed that each FPGI will take up to 2,000 hours, about one full-time equivalent position. Additional staff time may be needed for inspection followup and enforcement activities depending on the inspection results. Resources are available to complete the pilot program described in this paper and to conduct up to four FPGIs per year as part of a permanent FPGI program. If, at some time, the staff determines that it should conduct more than four inspections per year, it will revisit the resource implications.

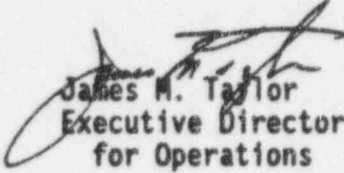
RECOMMENDATION:

The NRC staff will implement the FPGI pilot program described herein, unless directed otherwise by the Commission within 10 days from the date of this paper. (The staff needs to schedule the first pilot inspection shortly so that it can be conducted no later than the first quarter of calendar year 1997.) The staff estimates that it can complete the pilot program within a year without adversely affecting other high-priority fire protection work.

The staff will report to the Commission the results of the FPGI pilot program.

COORDINATION:

NRR is coordinating program development and inspection scheduling with the four regional offices and enforcement activities with OE.


James M. Taylor
Executive Director
for Operations

SECY NOTE: In the absence of instructions to the contrary, SECY will notify the staff on Monday, January 13, 1997 that the Commission, by negative consent, assents to the action proposed in this paper.

DISTRIBUTION:

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Typical Fire Protection Program Supporting Documentation

1. The current version of the Fire Protection Program and Fire Hazards Analysis.
2. Current versions of the fire protection program implementing procedures (e.g., administrative controls, surveillance testing, fire brigade).
3. Fire brigade training program and pre-fire plans.
4. Post-fire safe shutdown systems and separation analysis.
5. Post-fire alternative shutdown analysis.
6. Piping and instrumentation (flow) diagrams highlighting the components used to achieve and maintain hot standby and cold shutdown for fires outside the control room and those components used for those areas requiring alternative shutdown capability.
7. Plant layout and equipment drawings which identify the physical plant locations of hot standby and cold shutdown equipment.
8. Color-coded,, marked-up electrical raceway drawings which identify the routing of power and control circuits for those plant systems necessary to achieve and maintain safe shutdown.
9. Plant layout drawings which identify plant fire area delineation, areas protected by automatic fire suppression and detection, and the locations of fire protection equipment.
10. Plant layout drawings which identify the general location of the post-fire emergency lighting units.
11. Associated circuit analysis performed to assure the shutdown functions and alternative shutdown capability are not prevented by hot shorts, shorts to ground, or open circuits (e.g., analysis of associated circuits for spurious equipment operations, common enclosure, common bus).
12. Plant operating procedures which would be used and describe shutdown from inside the control room with a postulated fire occurring in any plant area outside the control room, procedures which would be used to implement alternative shutdown capability in the event of a fire in either the control or cable spreading room.
13. Maintenance and surveillance testing procedures for alternative shutdown capability and fire barriers, detectors, pumps and suppression systems.
14. Maintenance procedures which routinely verify fuse breaker coordination in accordance with the post-fire safe shutdown coordination analysis.

ENCLOSURE 2

15. A sample of significant fire protection and post-fire safe shutdown related design change packages (including their associated 10 CFR 50.59 evaluations) and Generic Letter 86-10 evaluations.
16. The reactor plant's IPEEE, results of any post-IPEEE reviews, and listings of actions taken/plant modifications conducted in response to IPEEE information.
17. Temporary modification procedures.
18. Organization charts of site personnel down to the level of fire protection staff personnel.
19. If applicable, layout/arrangement drawings of potential reactor coolant/recirculation pump lube oil system leakage points and associated lube oil collection systems.