Wisconsin Electric
POWER COMPANY

Point Beach Nuclear Plant
6610 Nuclear Rd., Two Rivers, WI 54241

(414) 755-2321

PBL 97-0058

February 14, 1997

Document Control Desk
US NUCLEAR REGULATORY COMMISSION
Mail Station P1-137
Washington, DC 20555

Ladies/Gentlemen:

DOCKET 50-266
LICENSEE EVENT REPORT 97-005-00
1SI-852A NOT TESTED IN ACCORDANCE
WITH TECHNICAL SPECIFICATIONS
POINT BEACH NUCLEAR PLANT, UNIT 1

Enclosed is Licensee Event Report 97-005-00 for Point Beach Nuclear Plant, Unit 1. This report is provided in accordance with 10 CFR 50.73(a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications." This report describes the identification of core deluge valve 1SI-852A as not being tested in accordance with the Inservice Test Program and Technical Specification 15.4.5.II.A.1.

Please contact us if you require additional information.

Sincerely,

for

Douglas F. Johnson
Manager - Regulatory Services
and Licensing

DAW
Enclosure

250133

IE22

cc: NRC Resident Inspector
NRC Regional Administrator

NRC FORM 366 (4-95) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 04/30/98

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Point Beach Nuclear Plant, Unit 1	05000266	1 OF 3

TITLE (4)

1SI-852A Not Tested In Accordance With Technical Specifications

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
										05000
01	16	97	97	-- 005 --	00	02	14	97	FACILITY NAME	DOCKET NUMBER 05000

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)	90		20.2201(b)		20.2203(a)(2)(v)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)
			20.2203(a)(1)		20.2203(a)(3)(i)		50.73(a)(2)(ii)		50.73(a)(2)(x)
			20.2203(a)(2)(i)		20.2203(a)(3)(ii)		50.73(a)(2)(iii)		73.71
			20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		OTHER
			20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		Specify in Abstract below
			20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)		or in NRC Form 366A

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
David Weaver	(414) 221-3418

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On January 16, 1997, while Point Beach Nuclear Plant (PBNP), Unit 1 was operating at 90 percent power, plant personnel determined that core deluge Valve 1SI-852A had not been tested in accordance with the Inservice Test Program as required by Technical Specification 15.4.5.II.A.1. Also since the valve was not stroke tested within 24 hours as stated in Specification 15.4.0.3 due to extended test preparation time, we were subsequently required to enter Specification 15.3.3.A.3.c. Specification 15.3.3.A.3.c allows any valve in the safety injection and residual heat removal systems, which is required to function during accident conditions, to be inoperable for up to 72 hours before placing the reactor in the hot shutdown condition. Valve 1SI-852A was stroke tested on January 17, 1997, and verified to be operable.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Point Beach Nuclear Plant, Unit 1	05000266	97	- 005 -	00	2 OF 3

TEXT *(If more space is required, use additional copies of NRC Form 366A)* (17)

Event Description:

On January 16, 1997, while Point Beach Nuclear Plant (PBNP), Unit 1 was operating at 90 percent power, plant personnel determined that core deluge Valve 1SI-852A had not been tested in accordance with the Inservice Test (IST) Program as required by Technical Specification 15.4.5.II.A.1. Also since the valve was not stroke tested within 24 hours as stated in Specification 15.4.0.3 due to extended test preparation time, we were subsequently required to enter Specification 15.3.3.A.3.c. Specification 15.3.3.A.3.c allows any valve in the safety injection and residual heat removal systems, which is required to function during accident conditions, to be inoperable for up to 72 hours before placing the reactor in the hot shutdown condition. Valve 1SI-852A was subsequently stroke tested on January 17, 1997, and verified to be operable. This discrepancy was identified as part of Unit 2 Start-Up Commitment 10, "Review the inservice testing (IST) acceptance criteria for all IST valves to ensure that the IST acceptance criteria meets the design basis/accident analysis requirements," as committed to in our December 12, 1996, letter. This action item has since been completed.

Cause:

Evaluation of this condition revealed that Valve 1SI-852A had not been completely tested since April 9, 1994. Prior to April 1994, the valve had been stroke tested in accordance with Inservice Test Procedure IT-3A, "RHR Pump and Valve Tests in DHR Mode (Cold Shutdown), Unit 1." Procedure IT-3A includes stroke timing of Valve 1SI-852A which fulfills, in part, the requirements of the inservice testing program. Since April 1994, Valve 1SI-852A had been stroked during performance of IT-750, "RHR Pump RWST Suction Check Valve Test (Refueling), Unit 1," but not timed. Therefore, these later tests did not meet the requirements of the inservice testing program. We have not yet determined why the complete stroke tests have not occurred since April 1994.

Corrective Actions:

Valve 1SI-852A was stroke tested on January 17, 1997, and verified to be operable.

A complete review and verification of surveillance testing frequency requirements for ASME Section XI components will be performed by April 4, 1997.

The current methodology for scheduling and tracking ASME Section XI surveillance tests will be reviewed to identify the root cause and provide recommendations to prevent recurrence by April 1, 1997.

Reportability:

This Licensee Event Report is being submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications."

Safety Assessment:

Valve 1SI-852A had been routinely stroke tested (including stroke timing) prior to April 1994. The valve satisfactorily met all acceptance criteria during these tests. Since 1994, Valve 1SI-852A had routinely been stroke tested without stroke timing. Based on the successful tests performed prior to April 1994 and subsequent successful tests, the valve has been operable since April 1994. Therefore, the health and safety of the public and plant personnel were not impacted by this event.

Similar Occurrences:

The following reports describe events involving a missed surveillance/ sample:

LER	Title
266/97-003-00	Spare Containment Penetrations Not Leak Tested In Accordance With Technical Specifications
266/96-014-00	Steam Generator Blowdown Sample Not Performed in Accordance With Technical Specifications