



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

DOCKET NO. 50-213

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 66  
License No. DPR-61

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Connecticut Yankee Atomic Power Company (the licensee) dated May 2, 1983 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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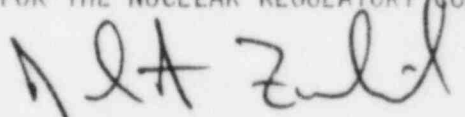
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C(2) of Facility Operating License No. DPR-61 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B as revised through Amendment No. 66, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Chief  
Operating Reactors Branch #5  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 3, 1985.

ATTACHMENT TO LICENSE AMENDMENT NO. 66

FACILITY OPERATING LICENSE NO. DPR-61

DOCKET NO. 50-213

Replace the following pages of the Appendix A Technical Specification with the enclosed pages as indicated. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the area of change.

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### 3.23 Post Accident Instrumentation

#### Applicability

This specification applies to specific plant instrumentation required for post-accident assessments.

#### Objective

To ensure the instrumentation will be operable if required.

#### Specification

##### A. Containment High Radiation Monitor

A.1 Two containment high radiation monitor channels shall be operable during modes 1, 2, 3, and 4 with:

- a) The capability to measure up to  $1 \times 10^6$  R/hr.
- b) The alarm setpoint less than or equal 100 R/hr.

A.2 With one or more channel(s) inoperable, restore the inoperable channel(s) to operable status within 7 days, or else prepare and submit a Special Report to the Commission pursuant to Specification 6.9.3 within the next 10 days outlining: the cause of the malfunction, the plans for restoring the channel to operable status, and any alternative methods for estimating containment dose rates during the interim.

B. Stack Wide Range Noble Gas Monitor

- B.1 One stack wide range noble gas monitor channel shall be operable during modes 1, 2, 3, and 4 with:
- a) The capability to measure up to  $1 \times 10^5$   $\mu\text{C/cc}$  of Xe-133.
  - b) The alarm setpoint less than or equal to the monitor reading corresponding to 1  $\mu\text{C/cc}$ .
- B.2 With less than one channel operable, return one channel to operable status within 7 days, or else prepare and submit a Special Report to the Commission pursuant to Specification 6.9.3 within the next 10 days outlining: the cause of the malfunction, the plans for restoring the channel to operable status, and any alternative methods for estimating stack release rates during the interim.

Basis

These specifications ensure the operability of the high range containment radiation monitors and wide range stack noble gas monitors. These monitors are required for post-accident assessments. This capability is consistent with the recommendations of Regulatory Guide 1.97, "Instrumentation for Light Water Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident", December 1980 and NUREG-0737, "Clarification of TMI Action Plan Requirements," November, 1980.

#### 4.16 Post Accident Instrumentation

##### A. Containment High Radiation Monitor

- A.1 Each containment high radiation monitor channel shall be demonstrated operable by:
- a) Performance of a channel check once per day.
  - b) Performance of a channel calibration and functional test once every refueling. Calibration of the sensor with a radioactive source need only be performed for levels less than or equal to 10R/hr. Higher levels may be calibrated electronically.

##### B. Stack Wide Range Noble Gas Monitor

- B.1 The stack wide range noble gas monitor channel shall be demonstrated operable by:
- a) Performance of a channel check once per day.
  - b) Performance of a channel calibration and functional test once every 92 days.

##### Basis

The test frequencies specified will ensure the operability of the containment and stack monitors.



## ADMINISTRATIVE CONTROLS

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### SPECIAL REPORTS

- 6.9.3 Special reports shall be submitted to the Regional Administrator of the Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:
- a. Inservice Inspection results, Specifications (4.10) and (4.12).
  - b. Primary Containment Leak Rate Results, Specification (4.4).
  - c. Reactor Vessel Material Surveillance Specimen Examination, Specification (4.10).
  - d. Steam Generator Tube Report

Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.

- e. Post-Accident Instrumentation Operability, Specification (3.23 A and B).

### 6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering the time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. Reports of all events which are required by regulations and Technical Specifications to be reported to the NRC in writing within 24 hours.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.