



GPU Nuclear, Inc.
U.S. Route #9 South
Post Office Box 388
Forked River, NJ 08731-0388
Tel 609-971-4000

February 12, 1997
6730-97-2048

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Dear Sir:

SUBJECT: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report - January, 1997

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1, enclosed are two (2) copies of the Monthly Operating Data (Gray Book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Ms. Brenda DeMerchant, Oyster Creek Regulatory Affairs Engineer, at 609-971-4642.

Very truly yours,

Michael B. Roche
Vice President & Director
Oyster Creek

MBR/BDeM/gl

Enclosures

cc: Administrator, Region I (2 copies)
NRC Project Manager
NRC Resident Inspector

IE2411

9702260047 970131
PDR ADOCK 05000219
R PDR

SUMMARY
JANUARY, 1997

The plant operated at full load until January 24, 1997 when power was reduced to perform quarterly MSIV testing. The plant returned to full power on January 26, 1997 following successful test completion, and remained at full power for the remainder of the month. During the month, the plant generated 470,346 MWh net electric which was 102.1% of its MDC rated capacity.

MONTHLY OPERATING REPORT

Licensee Event Reports - January, 1997

LER 96-014 was filed January 8, 1997

On November 13, 1996, it was discovered that mounting bolts were missing from a seismic restraint frame around Motor Control Center (MCC) DC-2. The MCC was declared inoperable and a Technical Specification LCO was entered. The four missing bolts were reinstalled and the MCC subsequently declared operable.

This condition was caused by personnel error. The seismic restraint was disassembled to support recent modification work. A final MCC walkdown at the completion of the work overlooked the bolts not being installed.

The safety significance of this occurrence is minimal since the equipment powered by this MCC is not relied upon to prevent or mitigate the consequences of an accident.

Required Reading will be issued and job orders provided with specific instructions to prevent recurrence.

LER 96-015 was filed January 20, 1997

On December 20, 1996, during a review of a vendor service letter, it was identified that the thermal overloads for the starters for the Reactor Water Clean Up (RWCU) Isolation Valves were not sized for a revised Reactor Building environment following a High Energy Line Break (HELB) from the RWCU System. This condition is considered reportable under 10 CFR 50.73(a)(2)(ii). The root cause of this deficiency is that previous HELB analyses were performed without using the most conservative assumptions.

The safety significance of this condition has been determined to be minimal. The reactor is normally operated at 100% power. Above approximately 88% power, a reactor scram is received with any break size larger than approximately 50% of the cross sectional area of the pipe. Below 50% break size, the analysis is bounded (both temperature profile and offsite release) by existing analyses. Immediate corrective actions were taken to provide the operators with guidance on addressing the potential line break. The thermal overloads were bypassed to ensure the ability to isolate the system. A re-analysis for other potentially affected Safety Related equipment was commenced to ensure continued operability for required systems. Modifications to provide an automatic RWCU isolation on a line break will be evaluated for possible installation.

OPERATING DATA REPORT

OPERATING STATUS

1. Docket:	50-219
2. Reporting Period:	January, 1997
3. Utility Contact	John D. Dougher (609)-971-2130
4. Licensed Thermal Power (MWt):	1930
5. Nameplate Rating (Gross MWe):	687.5 x 0.8 = 550
6. Design Electrical Rating (Net MWe)	650
7. Maximum Dependable Capacity (Gross MWe):	641
8. Maximum Dependable Capacity (Net MWe):	619
9. If Changes Occur Above Since Last Report, Give Reasons:	None
10. Power Level to Which Restricted, If Any (Net MWe):	None
11. Reason For Restriction, If Any:	None

	<u>Month</u>	<u>Year</u>	<u>Cumulative</u>
12. Report Period Hours	744.0	744.0	237648.0
13. Hours RX Critical	744.0	744.0	162332.8
14. RX Reserve Shutdown Hours	0.0	0.0	918.2
15. Hours Generator On-Line	744.0	744.0	158661.8
16. UT Reserve Shutdown Hours	0.0	0.0	0.0
17. Gross Thermal Energy (MWH)	1415602	1415602	275329682
18. Gross Electric Energy (MWH)	487500	487500	92416767
19. Net Electric Energy (MWH)	470349	470349	88678915
20. UT Service Factor	100.0	100.0	66.8
21. UT Available Factor	100.0	100.0	66.8
22. UT Capacity Factor (MDC Net)	102.1	102.1	60.9
23. UT Capacity Factor (DER Net)	97.3	97.3	57.4
24. UT Forced Outage Rate	0.0	0.0	9.7
25. Forced Outage Hours	0.0	0.0	17137.4
26. Shutdowns Scheduled Over Next 6 Months (Type, Date, Duration)	None		
27. Currently Shutdown, Estimated Startup Date:	N/A		

Oyster Creek Station #1

Docket No. 50-219

REFUELING INFORMATION - JANUARY, 1997

Name of Facility: Oyster Creek Station #1

Scheduled date for refueling shutdown: September 1998

Scheduled date for restart following refueling: October 1998

Will refueling or resumption of operation thereafter require
a Technical Specification change or other license
amendment? No

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier,
unreviewed design or performance analysis methods, significant changes in fuel design, new operating
procedures:

1. General Electric Fuel Assemblies - Fuel design and performance analysis methods have been approved by the
NRC.

The number of fuel assemblies	(a) in the core	=	560
	(b) in the spent fuel storage pool	=	2236
	(c) in new fuel storage vault	=	8

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that
has been requested or is planned, in number of fuel assemblies:

Present Licensed Capacity: 2645

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present
licensed capacity:

Full core discharge capacity to the spent fuel pool was lost after the 1996 refueling outage.

DOCKET NO: 50-219
 UNIT NAME: Oyster Creek
 DATE: February 6, 1997
 COMPLT'D BY: David M. Egan
 TELEPHONE: 609/971-4818

REPORT MONTH: January 1997

No.	DATE	TYPE F: Forced S: Scheduled	DURATION (hours)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
1	970124	S	0	b	1	Power was manually reduced to perform quarterly MSIV testing.

SUMMARY:

(1) REASON

- a. Equipment Failure (Explain)
- b. Maintenance or Test
- c. Refueling
- d. Regulatory Restriction
- e. Operator Training & Lic Exam
- f. Administrative
- g. Operational Error (Explain)
- h. Other (Explain)

(2) METHOD

- 1. Manual
- 2. Manual Scram
- 3. Automatic Scram
- 4. Other (Explain)