



PSEG Public Service
Electric and Gas
Company

80 Park Plaza, Newark, NJ 07101 / 201 430-8217 MAILING ADDRESS / P.O. Box 570, Newark, NJ 07101

Robert L. Mittl General Manager
Nuclear Assurance and Regulation

September 4, 1985

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, MD 20814

Attention: Mr. Walter Butler, Chief
Licensing Branch 2
Division of Licensing

Gentlemen:

SAFETY EVALUATION REPORT CONFIRMATORY ISSUE 1
HOPE CREEK GENERATING STATION
DOCKET NO. 50-354

Pursuant to discussions with D. Wagner and R. Li, the following information is being provided on the stresses imposed on the feedwater check valve hinge pin following a double ended break of the feedwater line outside containment. A manual elastic analysis was performed for the hinge shaft (pin) housing and shaft using the maximum angular velocity generated in the hinge arm analysis. Since the stress levels for bending in the shaft housing and for shear in the shaft, as determined by this analysis, are considerably less than the allowable of 0.70 Su as specified by Appendix F, Table F-1322.2-1 of the ASME B&PV Code, Section III, it is concluded that the hinge shaft housing and shaft are acceptable.

Should you have any questions in this regard, please contact us.

Very truly yours,

R L Mittl / R Douglas

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PDR ADOCK 05000354
E PDR

The Energy People

Mr. Walter Butler

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9/4/85

C D. H. Wagner
USNRC Licensing Project Manager

A. R. Blough
USNRC Senior Resident Inspector

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