

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Washington Nuclear Plant - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 9 7										PAGE (3) 1 OF 0 3																																		
TITLE (4) RCIC Containment Isolation Valve Closure Time																																																						
EVENT DATE (6)										LER NUMBER (8)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																								
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES										DOCKET NUMBER(S)																	
0 8			0 1			8 5			8 5			0 5			2 0			0 0			8 3			0 8			5													0 5 0 0 0														
0 8			0 1			8 5			8 5			0 5			2 0			0 0			8 3			0 8			5													0 5 0 0 0														
OPERATING MODE (9) 1										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																												
POWER LEVEL (10) 0 7 1 0										20.402(b)										20.406(c)										80.73(a)(2)(iv)										73.71(b)														
										20.406(a)(1)(i)										80.36(c)(1)										80.73(a)(2)(v)										73.71(c)														
										20.406(a)(1)(ii)										80.36(c)(2)										80.73(a)(2)(vii)										OTHER (Specify in Abstract below and in Text, NRC Form 366A)														
										20.406(a)(1)(iii)										80.73(a)(2)(i)										80.73(a)(2)(viii)(A)																								
										20.406(a)(1)(iv)										80.73(a)(2)(ii)										80.73(a)(2)(viii)(B)																								
										20.406(a)(1)(v)										80.73(a)(2)(iii)										80.73(a)(2)(x)																								
LICENSEE CONTACT FOR THIS LER (12)																																																						
NAME																				TELEPHONE NUMBER																																		
W. S. Davison, Compliance Engineer																				AREA CODE 5 0 9 3 1 7 1 - 1 2 1 5 0 1 1																																		
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) Ext. 2726																																																						
CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NRCDS					CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NRCDS									
SUPPLEMENTAL REPORT EXPECTED (14)																																																						
YES (If yes, complete EXPECTED SUBMISSION DATE)																				NO										EXPECTED SUBMISSION DATE (15)																								
																				X																																		

Due to an error in transferring valve stroke time requirements, a procedure deviation was issued to a surveillance procedure which supplied an incorrect closure time criteria for RCIC-V-69. A subsequent performance of the procedure resulted in a measured time of 8.35 seconds which is in excess of the 8.0 second Technical Specification requirement. When the evaluation of closure time was made to the incorrect criteria, the violation of the Technical Specification requirement was not apparent. The next performance of the test, 20 days later, measured the closure time at 7.98 seconds. Approximately two months after the original deviation, the error was discovered and corrected.

8509050188 850830
PDR ADOCK 05000397
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Washington Nuclear Plant - Unit 2	01500039785	85	052	000	2	OF 3

TEXT (if more space is required, use additional NRC Form 305A's) (17)

Plant Conditions

- a) Power Level - 70.4%
- b) Reactor Mode - 1

Event

On 6/8/85 a procedure deviation was implemented to PPM 7.4.7.3.3, RCIC Operability Test, which, among other changes, incorrectly changed the closure time requirement for containment isolation valve RCIC-V-69 from 4 to 8 seconds to 4 to 10 seconds. The incorrect time in the procedure deviation was a result of incorrect copying of the correct value on the new table. Technical Specification Table 3.6.3-1 requires this valve to close in less than or equal to 8.0 seconds. The procedure deviation was approved by the Plant Operating Committee (POC) on June 26, 1985.

PPM 7.4.7.3.3 was performed on 6/28/85 and RCIC-V-69 closure time was measured at 8.35 seconds but the failure to meet the Technical Specification requirement was not recognized because of the incorrect criteria change.

PPM 7.4.7.3.3 was performed on 7/18/85 and the closure time was measured at 7.98 seconds.

On 8/1/85 the fact that the closure time requirement of RCIC-V-69 in PPM 7.4.7.3.3 was not in agreement with Technical Specifications was realized. The subsequent search of the surveillance document records revealed the 8.35 second test results of 6/28/85.

Immediate Corrective Action

A procedure deviation to PPM 7.4.7.3.3 was issued on 8/1/85 to return the RCIC-V-69 closure criteria to the correct value. A search of documentation was made on 8/1/85 which determined that the most recent RCIC-V-69 closure time of 7.98 seconds met the 8 second Technical Specification requirement.

Further Corrective Action

- o This is a singular occurrence and Plant personnel involved with this deviation have been cautioned to ensure correctness of information in procedure deviations.
- o RCIC-V-69 has a Technical Specification closure time requirement but does NOT have any automatic isolation logic so that if an isolation of that valve is ever required, it will only occur when its control switch is manually placed in the close position. Thus, a change request will be developed and submitted to the Technical Specifications to delete stroke times for this and other valves which do not receive automatic actuation signals.

ISV

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

August 30, 1985

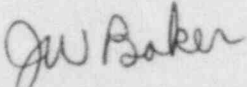
Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 85-052

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-052 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,



C.M. Powers (M/D 927M)
WNP-2 Plant Manager

CMP:1a

Enclosure:

Licensee Event Report No. 85-052

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

IE22
11