

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) D. C. COOK NUCLEAR PLANT, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 1 1 6	PAGE (3) 1 OF 0 1
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TITLE (4)

ESF ACTUATION

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 8	0 3	8 5	8 5	0 1	5	0 8	3 0	8 5			0 5 0 0 0

OPERATING MODE (9) 4	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
	20.402(b)		20.405(c)		<input checked="" type="checkbox"/> 50.73(a)(2)(iv)		73.71(b)			
	20.405(a)(1)(i)		50.38(c)(1)		50.73(a)(2)(v)		73.71(c)			
	20.405(a)(1)(ii)		50.38(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
	20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)					
POWER LEVEL (10) 0 0 0	20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
	20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)								TELEPHONE NUMBER			
K. R. BAKER - OPERATIONS SUPERINTENDENT								AREA CODE		6 1 6	
										4 6 5 - 5 9 0 1	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		

SUPPLEMENTAL REPORT EXPECTED (14)								EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)								V A				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

ON AUGUST 3, 1985 AT 2250 HOURS WITH THE REACTOR COOLANT SYSTEM IN MODE 4 (HOT SHUTDOWN), AN ESF ACTUATION (FEEDWATER ISOLATION) OCCURRED AS A RESULT OF A HIGH-HIGH FEEDWATER LEVEL IN THE LOOP 4 STEAM GENERATOR. THE LEVEL DEVIATION WAS DUE TO EXCESSIVE LEAKAGE THROUGH THE LOOP 4 AUXILIARY FEEDWATER VALVE WHILE MANUALLY CONTROLLING FEEDWATER LEVEL.

IT WAS NOTED THAT THE LOOP 4 STEAM GENERATOR LEVEL WAS INCREASING RELATIVE TO THAT OF LOOP 1. THE LOOP 4 AUXILIARY FEEDWATER VALVE WAS CLOSED BECAUSE OF EXCESSIVE LEAKAGE THROUGH THE VALVE. THE UNIT SUPERVISOR INSTRUCTED THE OPERATOR RESPONSIBLE FOR STEAM GENERATOR FEEDWATER LEVEL CONTROL TO CYCLE THE VALVE IN AN ATTEMPT TO STOP THE LEAKAGE. THE ATTEMPT WAS UNSUCCESSFUL AND AT 2237 HOURS THE WEST AUXILIARY FEEDWATER PUMP (FEEDING BOTH LOOPS 1 AND 4 STEAM GENERATORS) WAS REMOVED FROM SERVICE. HOWEVER, THE EXISTING LEVEL INCREASE, COMBINED WITH THE RESULTING SWELL WITHIN THE STEAM GENERATOR, CAUSED TWO OF THE THREE LEVEL CHANNELS TO REACH THE HIGH-HIGH SETPOINT AND PRODUCE THE FEEDWATER ISOLATION.

SINCE THE UNIT WAS IN THE PROCESS OF COOLING DOWN, THE MAJOR EQUIPMENT THAT WOULD HAVE ACTUATED DUE TO THIS INCIDENT HAD BEEN REMOVED FROM SERVICE. CONSEQUENTLY, NO AUTOMATIC ACTUATIONS OCCURRED.

TO PREVENT RECURRENCE THE LOOP 4 AUXILIARY FEEDWATER VALVE (FMO-242) HAS BEEN REMOVED FROM SERVICE AND WILL BE REPAIRED PRIOR TO UNIT STARTUP.

B509040416 B50830
PDR ADOCK 05000316
S PDR

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INDIANA & MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106
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August 30, 1985

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

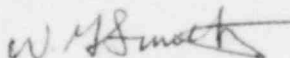
Operating License DPR-74
Docket No. 50-316

Document Control Manager:

In accordance with the criteria established by 10CFR50.73
entitled Licensee Event Reporting System, the following
report/s are being submitted:

RO 85-015-0

Sincerely,


W.G. Smith, Jr.
Plant Manager

/cbm

Attachment

cc: John E. Dolan
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