

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) D.C. COOK NUCLEAR PLANT, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5	PAGE (3) 1 OF 0 1
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TITLE (4)

ESF ACTUATIONS

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)												
0	8	0	3	8	5	8	5	0	3	7	0	0	0	8	3	0	8	5	0	5	0	0	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

OPERATING MODE (9) 5	20.402(b)	20.405(c)	X	50.73(a)(2)(iv)	73.71(b)
	20.405(a)(1)(i)	50.38(c)(1)		50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.38(c)(2)		50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 386A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(vii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)	
POWER LEVEL (10) 0 0 0	20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME A.A. BLIND - ASSISTANT PLANT MANAGER	TELEPHONE NUMBER 6 1 6 4 6 5 - 5 9 0 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ON AUGUST 3, 1985, AT 2339 HOURS, AND AGAIN ON AUGUST 7, 1985, AT 0042 HOURS WITH THE UNIT IN MODE 5 (COLD SHUTDOWN), AUTOMATIC ESF ACTUATIONS OCCURRED WHICH WERE DIRECTLY RELATED TO THE INSTALLATION OF AN APPROVED DESIGN CHANGE (RFC-2827). THIS REPORT IS BEING SUBMITTED SOLELY BECAUSE THE CONTROL ROOM OPERATORS WERE NOT NOTIFIED THAT AN ESF ACTUATION WAS IMMINENT DUE TO THE NATURE OF THE WORK TO BE PERFORMED.

THE AUGUST 3 ACTUATION CONSISTED OF A SAFETY INJECTION SIGNAL, IT RESULTED FROM SIMULTANEOUSLY TRIPPING 2 BISTABLES FOR THE COMPLETION OF THE ABOVE MENTIONED DESIGN CHANGE (ONE FOR THE CALIBRATION OF A TRANSMITTER AND THE OTHER FOR THE INSTALLATION OF A TRANSMITTER).

THE AUGUST 7 ACTUATION WAS A STEAM LINE ISOLATION SIGNAL, AND WAS THE RESULT OF THE FOLLOWING THREE CONDITIONS EXISTING SIMULTANEOUSLY; 1) A STANDING HIGH STEAM LINE FLOW FOR LOOP 1 (DUE TO TRANSMITTER REPLACEMENT FOR DESIGN CHANGE, 2) A HIGH STEAM LINE FLOW FOR LOOP 3. THIS INPUT WAS GENERATED WHEN A STEAM FLOW TRANSMITTER ISOLATION VALVE LEAKED DURING AN ONGOING HYDROSTATIC TEST, AND 3) A LOW STEAM GENERATOR PRESSURE SIGNAL (A NORMAL CONDITION FOR MODE 5).

SINCE THE UNIT WAS IN COLD SHUTDOWN DURING BOTH INCIDENTS AND NO MAJOR EQUIPMENT WAS ACTUATED, IT IS CONCLUDED THAT THERE WERE NO ADVERSE SAFETY CONSEQUENCES BECAUSE OF THESE EVENTS.

IN AS MUCH AS EITHER EVENT WOULD NOT HAVE OCCURRED HAD IT NOT BEEN FOR THE INSTALLATION OF THE DESIGN CHANGE, NO ACTION IS PLANNED TO PREVENT RECURRENCE.

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PDR ADOCK 05000315
PDR



INDIANA & MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106
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August 30, 1985

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Operating License DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10CFR50.73
entitled Licensee Event Reporting System, the following
report/s are being submitted:

RO 85-037-0

Sincerely,

W.G. Smith, Jr.
Plant Manager

/cbm

Attachment

cc: John E. Dolan
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11