

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Duane Arnold Energy Center										DOCKET NUMBER (2) 0 5 0 0 0 3 3 1										PAGE (3) 1 OF 0 2																																																	
TITLE (4) Failure to Perform Required Surveillance in Neutron Noise Region																																																																					
EVENT DATE (5) MONTH DAY YEAR 0 8 0 2 8 5										LER NUMBER (6) YEAR SEQUENTIAL NUMBER REVISION NUMBER 5 8 5 0 3 2 0 0 0										REPORT DATE (7) MONTH DAY YEAR 8 3 0 8 5										OTHER FACILITIES INVOLVED (8) FACILITY NAMES DOCKET NUMBER(S) None 0 5 0 0 0																																							
OPERATING MODE (9) N										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																																											
POWER LEVEL (10) 0 5 0										20.402(b) 20.406(a)(1)(i) 20.406(a)(1)(ii) 20.406(a)(1)(iii) 20.406(a)(1)(iv) 20.406(a)(1)(v)										20.406(e) 50.38(a)(1) 50.38(a)(2) 50.73(a)(2)(i) 50.73(a)(2)(ii) 50.73(a)(2)(iii)										50.73(a)(2)(iv) 50.73(a)(2)(v) 50.73(a)(2)(vi) 50.73(a)(2)(viii)(A) 50.73(a)(2)(viii)(B) 50.73(a)(2)(ix)										73.71(b) 73.71(a) OTHER (Specify in Abstract below and in Text, NRC Form 306A)																													
LICENSEE CONTACT FOR THIS LER (12) NAME Kenneth S. Putnam, Technical Support Engineer																														TELEPHONE NUMBER AREA CODE 3 1 9 8 5 1 - 7 4 5 6																																							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																																					
<table border="1"><thead><tr><th>CAUSE</th><th>SYSTEM</th><th>COMPONENT</th><th>MANUFACTURER</th><th>REPORTABLE TO NPROS</th><th>CAUSE</th><th>SYSTEM</th><th>COMPONENT</th><th>MANUFACTURER</th><th>REPORTABLE TO NPROS</th></tr></thead><tbody><tr><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td></tr><tr><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td></tr><tr><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td><td></td></tr></tbody></table>																														CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS																														
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SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) NO																				EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR																																																	

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At approximately 2400 hours on August 2, 1985, while reducing power for maintenance activities, the Duane Arnold Energy Center entered a region of power operation in which the load line was in excess of 80% of rated power and core flow was less than 45% of full flow. Recently revised plant Technical Specifications require that the Average Power Range Monitor (APRM) and Local Power Range Monitor (LPRM) neutron noise levels be determined within 2 hours of entering this region of operation. Contrary to this requirement the plant operated for a period in excess of seven hours without performing the required surveillance.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) Duane Arnold Energy Center	DOCKET NUMBER (2) 0 5 0 0 0 3 3 1	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A.) (17)

At approximately 2400 hours on August 2, 1985, while reducing power for maintenance activities, the Duane Arnold Energy Center entered a region of power operation in which the load line was in excess of 80% of rated power and core flow was less than 45% of full flow. Recently revised plant Technical Specifications require that the Average Power Range Monitor (APRM) and Local Power Range Monitor (LPRM) (EIIS system JC) neutron noise levels be determined within 2 hours of entering this region of operation. Contrary to this requirement the plant operated for a period in excess of seven hours without performing the required surveillance.

Plant Technical Specification 3.3.E was revised for Cycle 8 operation in response to General Electric Service Information Letter 380, Rev. 1, which discussed the possibility of core thermal hydraulic instability. The required surveillance is intended to show that power fluctuations associated with core thermal hydraulic instability are not occurring. Neutron flux oscillations are not a major safety concern, as the APRM scram on high flux will be reached prior to exceeding levels where fuel damage may occur. In addition, these oscillations are easily recognized and remedial action by the operators would be taken to correct the instability. A review of recorded plant information for the period in which the surveillance was required reveals that no core thermal hydraulic instability existed.

Although no physical plant parameter (i.e., core thermal hydraulic instability) existed which is prohibited, this event is being reported under 50.73(a)(2)(i)(B) as, "operation prohibited by the plant's Technical Specifications," because an action statement requiring surveillance performance was not complied with in accordance with the limiting condition for operation.

The cause of the missed surveillance was operator error. The period in question was the first time the plant had entered this region of operation since the implementation of the revised Technical Specification. The oversight was discovered when the next Operations shift came on duty. Upon identification of the oversight, Operations personnel promptly initiated activity for control rod insertions to remove the reactor from the surveillance region of operation. Shortly after 0700 hours on August 3, 1985 reactor power was reduced to outside the surveillance region. At 1241 hours on August 3, 1985 the plant intentionally entered the surveillance region again to establish baseline data for neutron noise. The test results were well within expected values.

In response to the error, plant management emphasized in writing to all Operations shift supervisors the importance of thoroughly reviewing all power maneuvers to ensure Technical Specification compliance.

Further, plant management directed that a special review of this and other recent operational deficiencies be conducted to comprehensively address appropriate additional corrective action to prevent recurrence.

Iowa Electric Light and Power Company

August 30, 1985
DAEC-85- 763

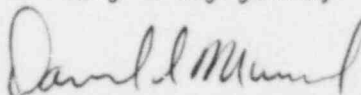
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Subject: Duane Arnold Energy Center
Docket No. 50-331
Op. License DPR-49
Licensee Event Report No. 85-032

Gentlemen:

In accordance with 10 CFR 50.73 please find attached a copy of the
subject Licensee Event Report.

Very truly yours,



Daniel L. Mineck
Plant Superintendent - Nuclear
Duane Arnold Energy Center

DLM/KSP/kp

attachment

cc: Mr. James G. Keppler
Regional Administrator
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

NRC Resident Inspector - DAEC

File A-118a

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