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November 15, 1996

Docket No. 50-461

10CFR50.90

Document Control Desk  
Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: Response to Follow-up Questions on Clinton  
Power Station Proposed Amendment of  
Facility Operating License No. NPF-62 (NS-96-004)

Dear Madam or Sir:

By letter dated August 15, 1996, [Illinois Power (IP) letter U-602624], IP submitted an application for amendment of Facility Operating License No. NPF-62, Appendix A - Technical Specifications, for Clinton Power Station. The proposed amendment, currently under review by the NRC staff, consists of a change to the Technical Specifications to incorporate a revised Safety Limit Minimum Critical Power Ratio (SLMCPR) calculated by General Electric (GE) for Cycle 7. On October 28, 1996, IP submitted a second letter (IP letter U-602651) which provided the formal response to a number of specific questions resulting from the NRC review of the proposed amendment.

In continued support of the NRC review of the proposed amendment, IP was requested to respond to a follow-up NRC question. On November 8, 1996, IP participated in a conference call with the NRC and GE to address a question from an NRC reviewer concerning the differences in uncertainties used for the SLMCPR statistical analyses performed for dual and single recirculation loop operation. As follow-up to the conference call, IP and GE agreed to provide a formal response to the NRC documenting the resolution of this issue. The following provides the requested information.

As discussed in IP letter U-602624, the CPS SLMCPR for Cycle 7 was calculated to be 1.09 for dual loop operation and 1.10 for single loop operation. These values were determined by GE using the analysis basis documented in GESTAR II (NEDE-24011-P-A-13), with the exceptions noted in the proposed license amendment. The analysis for single loop operation was performed in a manner similar to that for dual loop operation except that the uncertainties assumed for total core flow and Traversing Incore Probe (TIP) readings were larger.

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Section 4.3.1.1.1 of GESTAR II states that the uncertainties used in the statistical analyses are presented in the GE Fuel Bundle Designs document (NEDE-31152P, Revision 5). Table 3-3 of this document lists the uncertainties for the total core flow and TIP readings for reload cores as follows:

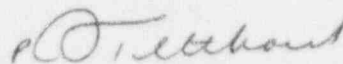
	Dual Loop Operation Standard Deviation (% of point)	Single Loop Operation Standard Deviation (% of point)
Total Core Flow	2.5	6.0
TIP Readings*	8.7	9.1

\* The values for TIP readings are root-mean-square combinations of the total uncertainty due to power distribution and TIP location (8.6%), and TIP reproducibility error during either dual loop operation (1.2%) or single loop operation (2.85%).

As stated above, these uncertainties were used in the cycle-specific SLMCPR calculations performed by GE for CPS Cycle 7.

As noted in the August 15, 1996 IP letter, the proposed amendment is required to support startup from the sixth (i.e., the current) refueling outage which began on October 13, 1996. As such, IP again respectfully requests review and approval of the subject amendment in a timeframe that supports that requirement.

Sincerely yours,



Paul J. Telthorst  
Director-Licensing

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cc: NRC Clinton Licensing Project Manager  
NRC Resident Office, V-690  
Regional Administrator, Region III, USNRC  
Illinois Department of Nuclear Safety