

August 22, 1985

Docket No. 50-334

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Mr. J. J. Carey, Vice President
Duquesne Light Company
Nuclear Group
Post Office Box 4
Shippingport, PA 15077

Dear Mr. Carey:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - NUREG-0737,
ITEM II.D.1 - PERFORMANCE TESTING OF RELIEF AND
SAFETY VALVES (LICENSING ACTION TAC 44562)

By letter dated January 10, 1985 you provided additional information on the subject matter for Beaver Valley Unit 1. We have reviewed that submittal, as well as submittals dated June 24, 1983, January 5, 1983, July 1, 1982 and March 29, 1982. We found that additional information is needed to complete our review, as described in the Enclosure.

Please respond to this Request For Additional Information within 45 days of receipt of this letter. If the target date offers any scheduling difficulty, or if you need any clarification, please feel free to contact your Project Manager, Mr. Peter Tam.

The reporting and/or recordkeeping requirements of this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,

/s/SVarga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As stated

cc:
See next page

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Mr. J. J. Carey
Duquesne Light Company

cc:

Mr. W. S. Lacey
Station Superintendent
Duquesne Light Company
Beaver Valley Power Station
Post Office Box 4
Shippingport, Pennsylvania 15007

Mr. K. Grada, Superintendent
of Licensing and Compliance
Duquesne Light Company
Post Office Box 4
Shippingport, Pennsylvania 15077

Mr. John A. Levin
Public Utility Commission
Post Office Box 3265
Harrisburg, Pennsylvania 17120

Gerald Charnoff, Esquire
Jay E. Silberg, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, DC 20036

Karin Carter, Esquire
Special Assistant Attorney General
Bureau of Administrative Enforcement
5th Floor, Executive House
Harrisburg, Pennsylvania 17120

Marvin Fein
Utility Counsel
City of Pittsburgh
313 City-County Building
Pittsburg, Pennsylvania 15219

Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 298
Shippingport, Pennsylvania 15077

Department of Environmental Resources
ATTN: Director, Office of
Radiological Health
Post Office Box 2063
Harrisburg, Pennsylvania 17105

Beaver Valley 1 Power Station

Mr. Thomas J. Czerpah
Mayor of the Burrough of
Shippingport
Post Office Box 26
Shippingport, Pennsylvania 15077

Pennsylvania Power Company
Ray E. Sempler
One E. Washington Street
New Castle, Pennsylvania 16103

Ohio Environmental Protection Agency
Division of Planning
Environmental Assessment Section
Post Office Box 1049
Columbus, Ohio 43216

Office of the Governor
State of West Virginia
Charleston, West Virginia 25305

Charles A. Thomas, Esquire
Thomas and Thomas
212 Locust Street
Box 999
Harrisburg, Pennsylvania 17108

Irwin A. Popowsky, Esquire
Office of Consumer Advocate
1425 Strawberry Square
Harrisburg, Pennsylvania 17120

Governor's Office of State Planning
and Development
ATTN: Coordinator, Pennsylvania
State Clearinghouse
Post Office Box 1323
Harrisburg, Pennsylvania 17120

Mr. Jess T. Shumate, Commissioner
State of West Virginia Department
of Labor
1900 Washington Street, East
Charleston, West Virginia 25305

Duquesne Light Company

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Beaver Valley 1 Power Station

cc:

David K. Heydinger, M.D.
State Director of Health
State Department of Health
1800 Washington Street, East
Charleston, West Virginia 25305

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Mr. Thomas M. Gerusky, Director
Bureau of Radiation Protection
Pennsylvania Department of
Environmental Resources
P.O. Box 2063
Harrisburg, Pennsylvania 17120

UNRESOLVED QUESTIONS ON
Beaver Valley Unit 1, NUREG 0737, II.D.1

1. Some information received indicates Beaver Valley Unit 1 uses Limitorque SMB-000-15 actuators with the PORV block valves. The EPRI test report submitted June 1, 1982 shows the actuators are Limitorque SMB-00-15. Clarify which actuator is used at Beaver Valley Unit 1.
2. In response to question 8 of our request for additional information, it was stated the PORV air supply pressure was increased to 60 psig and the air supply tubing size was increased, but the response did not state what new tubing size was used. Provide the size of the new tubing installed at the plant. Also the response stated that, after the modifications were made, testing of the PORVs resulted in acceptable stroke times being achieved. The answer did not indicate what these stroke times were. Provide the results of the stroke time testing for our review.
3. What is the torque setting of the Limitorque PORV block valve operator in the plant and what torque does the operator produce at this setting?
4. In response to question 3 of our request for additional information, reference was made to a Westinghouse Owners Group analysis which showed valve blowdown up to 10% did not result in the pressurizer filling nor were there any adverse effects on plant safety. Provide a copy of this report for our review.
5. In the piping analysis report submitted June 24, 1983, it was stated the proper functioning of the insulated boxes enclosing the SRV loop seals would be verified by testing. Provide the results of the testing for our review to confirm the SRV loop seal temperatures are now in the range of 310°F.

6. The piping analysis also indicated modifications would be made to one snubber and some anchor bolts would be proof-tested to higher loads than used during their original installation. Were the snubber modification and the results of the new anchor bolt proof testing included in the piping analysis performed after EPRI testing? In addition, after the anchor bolt testing is completed, provide a comparison of the allowable and calculated loads for those bolts tested.
7. Your response to our request for additional information stated the UFSAR analysis of the Feedwater Line Break (FWLB) showed the SRVs passed liquid for 1600 s but the Target Rock 69C test valve was tested with liquid for only 900 s. Based on the 900 s of testing it was concluded the valve would perform acceptably under plant conditions. Explain how valve performance in the 900 s of EPRI tests can be extrapolated to the 1600 s of liquid flow based on the FWLB analysis. Since some valve damage was noted at the end of 900 s of testing, provide assurance the additional liquid flow calculated for the FWLB will not damage the valve to the point it will not reseal. Note: It was stated in the answer to question 1 of our request for additional information that the UFSAR analysis was conservative and liquid flow from the SRVs was not expected if more realistic assumptions were used with respect to various input parameters. While this is most likely true, it is the NRC's position that the analysis with the more realistic assumptions must be performed and the results provided for our review before we can accept the conclusion the SRVs will not pass liquid.