

For PDR

August 29, 1985

For: The Commissioners

From: T. A. Rehm, Assistant for Operations, Office of the EDO

Subject: WEEKLY INFORMATION REPORT - WEEK ENDING AUGUST 23, 1985

A summary of key events is included as a convenience to those Commissioners who may prefer a condensed version of this report.

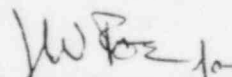
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*No input this week.

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PDR COMMS NRCC
WEEKLY INFOREPT PDR


T. A. Rehm, Assistant for Operations
Office of the Executive Director
for Operations

Contact:
T. A. Rehm, EDO
492-7781

HIGHLIGHTS OF WEEKLY INFORMATION REPORT

WEEK ENDING AUGUST 23, 1985

Shoreham

On August 15, 1985, at 8:40 pm, the Shoreham Plant was raising reactor power to about 2.5% to perform HPCI testing when one of the turbine bypass valves started to oscillate, causing a spike in reactor power. The licensee attempted to control the oscillation by adjusting the reactor pressure regulator, but two more oscillations occurred. The last one caused a power spike to approximately 5.5% power. The licensee has suspended testing until it can investigate the cause of the oscillations and correct it.

Meeting with the State of Virginia

On August 21, 1985, OSP and Region II staff members met with the Assistant Health Commissioner and staff in Richmond, Virginia. The State indicated its intention to proceed with full Agreement State status with NRC (excluding regulation of uranium milling) and also discussed its desire to enter into an MOU regarding inspection of low-level radioactive waste packaging and transportation at NRC licensed facilities.

Public Service Electric and Gas Company

On August 21, 1985, Public Service Electric and Gas Company was issued Materials License No. SNM-1953 for Hope Creek Generating Station, authorizing the receipt, possession, inspection, and storage of uranium enriched in the U-235 isotope contained in fuel assemblies in their shipping containers and spent fuel pool.

Readiness Review Program

On August 19-20, 1985, the Chief, Quality Assurance Branch, Division of Quality Assurance, Vendor, and Technical Training Programs participated in a meeting with Region II and NRR in Atlanta, GA to discuss lessons learned from NRC review of Module One of the Readiness Review Program so as to improve the review process for future modules.

OFFICE OF ADMINISTRATION

Week Ending August 23, 1985

ADMINISTRATION OF THE FREEDOM OF INFORMATION ACT

STATUS OF REQUESTS

	<u>Initial Request</u>	<u>Appeal of Initial Decision</u>
Carryovers, 1984	179	23
Received, 1985	588	33
Granted	460	25
Denied	126	13
Pending	181	18

ACTIONS THIS WEEK

Received

Rod Buckles, NUS Corporation (85-576)	Requests data regarding nuclear power plant declarations of "unusual events" and "alerts" in 1984.
Steven Aftergood, Committee to Bridge the Gap (85-577)	Requests copies of "Review of NRC's Reactor Safeguards Program" and "Follow-Up on OIA Report 'Review of NRC's Reactor Safeguards Program'".
Vincent J. Kiernan, The Herald (85-578)	Requests a copy of FOIA-80-589 from John Vail regarding Nuclear Fuel Services of Erwin, Texas.
Steven Aftergood, Committee to Bridge the Gap (85-579)	Requests all records regarding a meeting in February 1985 between representatives of the Israeli Atomic Energy Commission and NRC's RES, IP, NRR and IE offices.
David A. Repka, Bishop, Liberman, Cook, Purcell & Reynolds (85-580)	Requests records reflecting the NRC's current policy and criteria for determining when individuals and organizations requesting data under the FOIA are not required to pay fees for NRC processing of a request.
Bruce W. Stainbrook, BBL Microbiology Systems (85-581)	Requests a copy of the license for American Micro Scan of Mahevah, New Jersey.

CONTACT: J. M. Felton
492-7211

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Received, Cont'd

Stephen H. Hanauer, Technical Analysis Corporation (85-582)	Requests copies of Power Plant Examinations Results Summary Sheets for certain reactor operator and senior reactor operator examinations of 12 specified plants.
Donald Pay, Technical Information Project (85-583)	Requests records regarding low-level radioactive waste disposal in South Dakota and agreement state status for South Dakota.
Albert V. Carr, Duke Power Company (85-584)	Requests records related to Enforcement Action EA 84-93 being taken against Duke Power Company (Catawba nuclear power plant).
Stephanie Dematitis, Data Base Inc. (85-585)	Requests records regarding NRC's data processing centers.
Kay L. Dove (85-586)	Requests a copy of SECY-85-65 and any related records.
Steven Aftergood, Committee to Bridge the Gap (85-587)	Requests four categories of records regarding the University of Michigan Ford nuclear reactor and its conversion from highly enriched uranium (HEU) fuel to low enriched uranium (LEU) fuel.
Sandra Swan Travis, Data Services (85-588)	Requests copies of quotations from respondents to RFQ No. 503155 issued on February 21, 1985, for storage space at the environmental secure storage facility in the Washington, DC, area.
Jay E. Silberg, Shaw, Pittman, Potts & Trowbridge (85-A-33-85-493)	APPEAL TO THE COMMISSION for the release of the denied portions of SECY-85-80.

Granted

T.F. Sheehan, Gilbert/ Commonwealth, Inc. (85-218)	In response to a request for a copy of the contract and nonproprietary version of the proposal related to RFP RS-ORM-85-312 entitled, "Lease of a Data Base Management System and Supporting Integrated Software Packages," made available a copy of the requested proposal. Informed the requester that the contract is already available at the PDR.
Jim Pedro, NUS Corporation (85-465)	In response to a request for five specified summaries of the IDCOR/NRC Technical Exchange Meetings, made available three meeting summaries. Informed the requester that meeting summaries for two meetings could not be found.

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Granted, Cont'd

Steven Fagan,
Fort Worth Star-
Telegram
(85-477)

In response to a request for a copy of a completed NRC staff report on Region IV management decisions regarding the March 8, 1984, "T-shirt incident" at the Comanche Peak nuclear power plant, informed the requester that there is no completed report subject to this request.

Linda Smith,
S. Levy
Incorporated
(85-507)

In response to a request for a copy of SECY-84-13B, made available a copy of the requested paper.

Lyle Graber,
NUS Corporation
(85-508)

In response to a request for copies of NUREG/CR-0073 and SECY-85-30, made available a copy of NUREG/BR-0073 entitled, "Project Managers Handbook". Informed the requester that the disclosed portions of SECY-85-80 are already available at the PDR.

(Individuals
requesting
information)
(85-557)

In response to a request for copies of records pertaining to exposure to radiation in 1984 at the St. John's Regional Health Center in Missouri, made available five records.

Vincent Clark,
Delta Lighting
Corporation
(85-558)

In response to a request for copies of licenses issued to Stocker and Yale, Incorporated, made available 14 records.

Richard A. Sloan,
Phillip C.
Goldstick &
Associates, Ltd.
(85-565)

In response to a referral from the DOJ of one record relating to the industrial gas industry, made available a copy of the record.

Ophelia G. Williams,
J/R/A Associates
(85-571)

In response to a request for biographical information pertaining to an NRC employee, made available a copy of the requested record.

Ellyn R. Weiss,
Union of Concerned
Scientists
(85-572)

In response to a request for a copy of an ACRS letter to the Commission on July 17, 1985, on the ACRS position on Commission's policy on safety goals, made available a copy of the requested record.

Vincent J. Kiernan,
The Herald
(85-578)

In response to a request for a copy of FOIA-80-589 from John Vail, made available a copy of the FOIA request.

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Denied

Nina Bell,
Nuclear Information the receipt, analysis and denial of one or more "Show
and Resource Service Cause" petitions filed by Cynthia Stewart on behalf of
(84-665) Jacksonians for Livable Energy Policies (JULEP)
requesting actions on the Grand Gulf nuclear power plant,
made available 22 records. Informed the requester that
additional records subject to this request are already
available at the PDR. Denied one record in its entirety
compiled for law enforcement purposes. Denied portions
of one record compiled for law enforcement purposes and
release of which would tend to inhibit the open and
frank exchange of ideas essential to the deliberative
process. Denied portions of one record, release of which
would tend to inhibit the open and frank exchange of
ideas essential to the deliberative process.

Lynn Cunningham,
The Times-
Picayune
Publishing Corp.
(85-197)
In response to a request for copies of records written or
received by the NRC subsequent to the December 31, 1984,
OIA report on Region II and the Grand Gulf nuclear power
plant, made available one record. Denied 29 records in
their entirety, release of which would interfere with
ongoing investigations on Grand Gulf. Denied nine
records in their entirety, release of which would tend to
inhibit the open and frank exchange of ideas essential to
the deliberative process.

Sylvia Tognetti,
Christic Institute
(85-456)
In response to a request for copies of (a) comments of
the Commissioners in providing guidance to the staff on
developing recommendations to Congress for legislative
measures in response to the Supreme Court decision in
Silkwood v. Kerr-McGee Corp., and the Commission staff
comments in response to SECY-84-146A, and (b)
recommendations and proposed legislation prepared by the
Office of the General Counsel for presentation to the
99th Congress in response to the Supreme Court Silkwood
decision and on the subject of punitive damages, made
available two records. Denied six records in their
entirety, release of which would tend to inhibit the open
and frank exchange of ideas essential to the deliberative
process.

Jim Thomas,
NTEU President
(85-509)
In response to a request for records between officials
in the Division of Facilities and Operations Support and
officials in the Division of Security relating to theft
of NRC equipment, denied in their entirety, three records
containing information which constitutes advice,
opinions, and recommendations.

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Denied, Cont'd

Robert T. Simmons
(85-516)

In response to a request for biographical data on an NRC employee and Systematic Assessment of Licensee Performance (SALP) reports for ten plants, made available two records. Informed the requester that additional records subject to this request are already available at the PDR. Denied portions of one record, release of which would constitute a clearly unwarranted invasion of personal privacy.

Lyle Graber,
NUS Corporation
(85-523)

In response to a request for a copy of SECY-85-209, denied this paper in its entirety, release of which would tend to inhibit the open and frank exchange of ideas essential to the deliberative process.

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WEEKLY INFORMATION REPORT
DIVISION OF CONTRACTS
WEEK ENDING AUGUST 23, 1985

IFB ISSUED

IFB No.: RS-ORM-85-334

Title: "On-Call Maintenance for Microcomputers and Related Equipment"

Description: On-call maintenance including all necessary maintenance labor, documentation, repair parts, supplies, tools, test equipment, transportation and their related services for NRC microcomputers and related equipment.

Period of Performance: 2 years

Sponsor: Office of Resource Management

Status: IFB issued on August 21, 1985. Bids due on September 19, 1985.

RFP ISSUED

RFP No.: RS-OIE-85-167

Title: "General Course in Non-Destructive Examination Technology and Codes"

Description: The purpose of this project is to provide NRC personnel the detailed technical training in nondestructive examination (NDE) and the applicable codes necessary to enable them to conduct in-depth inspection of NRC licensed facilities under construction or modification to determine whether or not such facilities and components are constructed properly in accordance with applicable codes and standards.

Period of Performance: 3 years

Sponsor: Office of Inspection and Enforcement

Status: RFP issued on August 16, 1985. Proposals due September 16, 1985.

RFP No.: RS-OIE-86-170

Title: "Technical Assistance for Emergency Response"

Description: The purpose of this project is to provide technical, logistical and administrative assistance for the maintenance and functioning of the NRC Headquarters Operations Center. The contractor shall provide technical, logistical and administrative support to the NRC's Headquarters Operations Center to fulfill exercise requirements and meet unanticipated needs that arise during normal operation or during an incident within a very short time span.

Period of Performance: 1 year with two one-year options.

Sponsor: Office of Inspection and Enforcement

Status: RFP issued on August 20, 1985. Proposals due September 20, 1985.

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PROPOSALS UNDER EVALUATION

RFP No.: ADM-85-235

Title: "NRC Translation Services"

Description: The contractor will be required to furnish translation services for reports and other related material provided by NRC when issued by a formal work order.

Period of Performance: 2 years

Sponsor: Office of Administration

Status: RFP closed on August 19, 1985. Proposals forwarded to Source Evaluation Panel for review on August 20, 1985.

RFP No.: RS-NMS-85-008

Title: "Technical Assistance in Hydrogeology - Project A - Testing"

Description: The contractor shall provide technical assistance to the NRC in its review of DOE site investigations for a potential high level waste geologic repository.

Period of Performance: 2 years with 3 one-year options.

Sponsor: Office of Nuclear Material Safety and Safeguards

Status: Negotiations completed on August 21, 1985. Best and Final offers are due on August 26, 1985.

RFP No.: RS-NRR-85-054

Title: "Development of an NRC Simulation Facility Evaluation Program"

Description: The contract objectives are to develop, refine and validate criteria and methodology for the review of nuclear power plant simulation facilities; recommend size and make-up of a review team; and develop a schedule for conduct of review. Work will include a review of applicable literature; development of a methodology; testing this methodology through conduct of actual simulation facility reviews and preparation of detailed recommendations.

Period of Performance: 14 months with an option for an additional 6 months.

Sponsor: Office of Nuclear Reactor Regulation

Status: Best and Final offers received on August 19, 1985 and forwarded to Source Evaluators for review on August 19, 1985.

CONTRACTS AWARDED

IFB No.: NRC-10-85-252

Title: "Micrographic Services of Source Drawing Documents"

Description: The contractor shall provide services to the U.S. Nuclear Regulatory Commission to produce microfiche from source documents provided.

Period of Performance: 24 months

Sponsor: Office of Administration

Status: Fixed Price Requirements contract awarded to Microform, Inc. in the estimated amount of \$165,475.00 on August 22, 1985.

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CONTRACTS AWARDED (cont'd)

IFB No.: RS-ASB-85-401

Title: "Stenographic Reporting Services"

Description: Provide stenographic reporting services outside and inside the Washington, D.C. Metropolitan Area (excluding NRC Commission meetings held in the D.C. area).

Period of Performance: 2 years

Sponsor: Atomic Safety and Licensing Board Panel

Status: Fixed Price Requirements contract awarded to Ace-Federal Reporters, Inc. in the estimated amount of \$1,388,848.00 effective August 16, 1985, outside D.C. and October 1, 1985, inside D.C.

RFP No.: NRC-02-85-002

Title: "Technical Assistance for Design Reviews of High Level Nuclear Waste Geologic Repositories"

Description: The contractor shall be required to provide support services, including analyses, computations, engineering evaluations, citations to literature, and documentation thereof to assist the NRC to review, evaluate, and comment on the design of a geologic repository contained in the U.S. Department of Energy's (DOE) site characterization plans (SCP).

Period of Performance: 3 years

Sponsor: Office of Nuclear Material Safety and Safeguards

Status: Cost-Plus-Fixed-Fee contract awarded to Itasca Consulting Group, Inc. in the estimated amount of \$1,417,252.46 effective August 22, 1985.

ADMINISTRATIVE MATTERS

1. The protest filed by Software AG of North America on July 17, 1985, against award of Contract No. NRC-33-85-312 to Cullinet Software, Inc. for data base management system software was dismissed without prejudice by the General Services Board of Contract Appeals on August 9, 1985. Dismissal of the protest was subject to certain conditions including furnishing of copies of NRC documentation regarding evaluation and award of the contract and holding a conference to discuss relevant information. After review of the documentation, Software AG declined the need for the conference.
2. On August 16, 1985, Compucom Security, Inc. filed a protest with the General Services Board of Contract Appeals (GSBCA) against a revised Request for Quotation (RFQ) 0001. The RFQ was issued as a brand name or equal for ADP Software, Multifunction Expansion Boards. Compucom Security, Inc. contends that the specifications are unjustifiably restrictive and are in excess of the minimum needs of the NRC. The award has been suspended pending the results of the hearing before the GSBCA.

ENCLOSURE A

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OFFICE OF NUCLEAR REACTOR REGULATION

ITEMS OF INTEREST

Week Ending August 23, 1985

Davis-Besse

On August 14, 1985, NRR issued a 50.54(f) letter to Toledo Edison Company due to the June 9, 1985 loss of all feedwater event at Davis-Besse. Pursuant to 10 CFR 50.54(f), the licensee is to furnish, under oath or affirmation, no later than 30 days from the date of the letter, their plans and programs to satisfy NRC concerns. The plan and programs are to specify those actions to be completed prior to restart of Davis-Besse and include a schedule for any longer term actions. There were three pages of expressed NRC concerns in the letter which fell into three broad categories as follows:

- A. Completion of the investigation of the June 9, 1985 event, including analysis of the equipment failures, determination of the root causes, determination of the implications for other equipment, and completion or corrective actions.
- B. The plant-specific findings regarding this event including the concerns identified in the NRC Fact Finding Team Report, NUREG-1154.
- C. The programmatic and management issues that have contributed to this event and more generally to the recent performance of Davis-Besse.

The NRC letter then requests the licensee to reexamine programs initiated under their Performance Enhancement Program (PEP) in light of the response to this letter.

This 50.54(f) letter also supersedes the Confirmatory Action Letter issued June 10, 1985 by Region III as lead responsibility for NRC staff actions relating to facility restart has been transferred to NRR. As per the June 10, 1985 Confirmatory Action Letter, it remains the staff's understanding that Davis-Besse will not be restarted without NRC approval.

Diablo Canyon Unit 2

Diablo Canyon Unit 2 is expected to achieve criticality between Sunday, August 18 to late Monday, August 19, 1985. The low power testing program would be completed about August 29. Based on the above schedule, Region V will notify Mr. Denton by letter, about four days after criticality as to readiness for full power license.

ENCLOSURE B

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Dresden Unit 2

At 0021 CDT a site alert was called for Dresden Unit 2, when switchyard difficulties relating to Unit 1 resulted in a fault in the 138 KV switchyard and tripped an auxiliary transformer causing a perturbation which then caused a unit trip and main transformer trip, hence loss of offsite power. The diesel generators activated and picked up loads. A Group 1 isolation was manually initiated. Decay heat was removed with the isolation condenser and water level was stabilized by use of CRD pump 2A (2B unavailable - maintenance). Reactor pressure and water level were stabilized.

The fault was determined to be transformer #12. Switching allowed it to be isolated and power was returned to the main transformer. Onsite power was restored at approximately 0540 CDT. The alert was downgraded to an unusual event which was lifted at 0925 CDT.

A very slight amount of radiation was released onsite through the isolation condenser vent. This level of radioactivity is still being evaluated. The plant is currently at hot standby.

San Onofre Nuclear Generating Station, Unit No. 1

On Friday, August 9, 1985, the failure of a diaphragm caused a relief valve in the reactor coolant post-accident liquid sampling system to open. The PASS cubicle was then flooded with clean water normally used to flush the PASS lines after sampling. A sump pump was set for manual operation and thus did not operate. The flooding was discovered by an operator. As a result of the submergence, all post-accident liquid sampling system at San Onofre Unit 1 capabilities are inoperable. The licensee expects to restore the capability to take undiluted grab samples in four weeks.

Shoreham

On August 15, 1985, at 8:40 pm, the Shoreham Plant was raising reactor power to about 2.5% to perform HPCI testing when one of the turbine bypass valves started to oscillate, causing a spike in reactor power. The licensee attempted to control the oscillation by adjusting the reactor pressure regulator, but two more oscillations occurred. The last one caused a power spike to approximately 5.5% power. The licensee has suspended testing until it can investigate the cause of the oscillations and correct it.

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Seabrook

The joint owners of Seabrook Station, at a regularly scheduled meeting held on August 14, 1985, voted to increase project funding to a level of \$8 million per week effective September 1.

The increase from the \$5 million per week funding level not in place is conditioned on the lifting of a Public Service Company of New Hampshire (PSNH) spending restriction by the New Hampshire Public Utilities Commission (NHPUC). PSNH is currently limited to contributing no more than its share of a \$5 million per week funding level. The owners also voted that the \$8 million per week funding level will be retroactive to August 1, subject to the lifting of the spending restriction.

The increase in funding to \$8 million per week is being made to keep the project on target for operation before the end of 1986.

Catawba Units 1 and 2

On August 15, 1985, Unit 1 was operating at 94% power and Unit 2 was undergoing preoperational testing. While the Catawba Unit 1 operator was attempting to perform a routine operability test on the "B" diesel generator (DG), he actuated the wrong breaker which caused a blackout signal. The DG actuated and performed as designed. The Unit 1 "B" motor-driven AFW pump and the turbine-driven AFW pump started automatically due to the blackout signal but were immediately secured. A temporary loss of containment cooling water, caused by the loss of incoming bus, resulted in an increase in Unit 1 containment temperature and pressure. In addition, an outlet valve off the Unit 2 volume control tank (VCT) (powered from a bus being supplied from Unit 1) closed resulting in the VCT having no outlet. At that time, the Unit 2 primary system was water solid with the positive displacement (PD) charging pump aligned to take suction from the refueling water storage tank (RWST).

This condition was unnoticed by the Unit 2 operator who was away from the controls responding to a request from the Unit 1 operators to assist them in the blackout recovery. Both Unit 1 temperature and pressure returned to normal following a containment air release and restoration of containment cooling water. The Unit 1 bus was subsequently realigned and the DG secured. But, on the Unit 2 side, the primary system letdown to the VCT continued resulting in the filling and possible overpressurization of the VCT. The actual pressure attained is unknown. Relief path was available but it is not known if relief valve opened. Relief valve setting is being verified.

Unit 1 power level remained at 94% throughout the event. Please note that the Unit 2 VCT discussed above is a recent replacement of the one which was blown apart due to an overpressurization event on April 20, 1985.

ENCLOSURE B

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Diablo Canyon Unit 2

The Pacific Gas & Electric Company informed the NRC staff that Diablo Canyon Unit 2 achieved criticality at 2:16 am PDT, on August 20, 1985.

Arkansas Nuclear One, Unit No. 1

At 4:16 pm, Sunday, August 11, 1985, ANO-1 tripped on an anticipatory trip of loss of feedwater. The plant was at approximately 98% power at the time. The "B" Main Feedwater (MFW) pump tripped off on a signal of thrust bearing wear. The "A" MFW pump then tripped because of the Reactor Protection System received a signal of low pressure from the MFW pump "A." The Emergency Feedwater (EFW) system actuated normally. The operators manually initiated High Pressure Injection for about 30 seconds (150 to 200 gallons) when pressurizer level went below 40 inches (this is normal procedure). The licensee is repairing the MFW pump bearing, checking the pressure switch of the "B" MFW pump, and correcting a secondary leak in a flange connection of a hand hole of a steam generator. The licensee should be able to start up later today.

Hatch Units 1 and 2

Georgia Power has informed us, by letter dated August 16, 1985 that its equipment qualification schedule for Hatch Primary Containment H₂ and O₂ Gas Analyzers may slip beyond the current November 30, 1985 deadline.² If Georgia Power requests a schedule extension beyond November 30, 1985, the extension will have to be approved by the Commission. It has encountered a delay in LOCA qualification testing, by COMSIP Inc., of the heat tracing on the tubing which feeds these Gas Analyzers. Georgia Power states that it expects that other BWR licensees whose equipment is being tested by COMSIP Inc. may be similarly affected.

Limerick Generating Station, Unit 1

On August 16, 1985, an "Order Suspending Operation Above 5 Percent Power" was issued for Limerick Generating Station, Unit 1, on the basis of the stay by the United States Court of Appeals for the Third Circuit of the Commission's Order which authorized issuance of Facility Operating License No. NPF-39. The court lifted its stay on August 21, 1985 and, accordingly, a "Rescission of Order" has been issued by the Acting Director, Hugh L. Thompson, of NRR.

At 9:00 am on August 22, the plant was at 15 percent and proceeding to a plateau of 22 percent of full power. With sufficient water for six weeks operations, all the planned tests at this level can be completed.

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Peach Bottom Atomic Power Station, Unit 3

Earlier this week, we were informed by both the Region and Philadelphia Electric Company that the current pipe inspections being performed as the result of Generic Letter 84-11 have uncovered at least 12 welds with crack indications. The majority of these welds had been previously examined in 1983 and found to be without cracks. They were then given induction heat stress improvement (IHSI) and again UT examined and found to be without indications. It has now been determined that at least one weld (10-02 20-inch RHR line) has a 360 degree circumferential crack approximately 35-55% thru-wall. The current Unit 3 inspections are relying primarily upon the new G.E. automated UT detection system called "SMART." It would appear that this new system has been able to detect cracks which has previously been missed in prior inspections. This issue of the quality of UT pipe crack inspections has also been raised by the staff after recent review of data resulting from the closer laboratory examinations of welds which had been obtained from various recirculation pipe replacement programs (i.e., evidence of missed crack indications).

A meeting with the BWR Owners' Group has been scheduled for the week of September 16, 1985 to discuss the issue of inspections as well as the status and direction of various industrial sponsored programs.

St. Lucie Plant, Unit No. 1

Florida Power and Light Company informed the staff on August 21, 1985 that Exxon Nuclear Corporation (ENC) had found an input error in its large break LOCA analysis for St. Lucie Plant, Unit No. 1. The error resulted in a calculated peak clad temperature that exceeds the limit of 2200°F established in 10 CFR 50.46. Initial results indicate that this limit could be exceeded by as much as 250°F at a linear heat rate of 15.0 kw/ft. Earlier this year a code error was discovered by EXXON that led to a reanalysis for St. Lucie 1 that predicted a peak clad temperature of 2187°F for the large break LOCA.

In doing the original large break LOCA analysis for St. Lucie 1, ENC used assumed pump data as input to the codes, in particular, the torque curve. When the actual pump data and associated torque curve were applied, the peak clad temperature increased and exceeds the 2200°F limit. The initial assessment indicates that a reduction in the linear heat generation rate setpoint from 15.0 kw/ft to 13.4 kw/ft will permit continued operation within the 2200°F limit of 10 CFR 50.46. The licensee has changed the setpoint to 13.4 kw/ft and, in addition, has voluntarily reduced power to 95% to provide a slightly greater margin of safety and provide some degree of flexibility.

The staff was advised that the other PWRs that might be affected have been reviewed and that actual pump data was used in each case and, therefore, are not affected by the problem found at St. Lucie 1.

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Cooper Nuclear Station

Control rod withdrawal began at Cooper Nuclear Station during the afternoon of August 20, 1985, ending an 11-month refueling and maintenance outage. During the outage which began September 16, 1984, all reactor coolant piping susceptible to intergranular stress corrosion cracking (IGSCC) was replaced with IGSCC-resistant material, and the reactor was refueled for Cycle 10 operation. In addition to the refueling and pipe replacement, other major plant modifications were performed as part of the Appendix R, equipment qualification and NUREG-0737, Supplement 1 programs at Cooper Station.

After startup, the plant will undergo an extensive series of tests during power ascension. Return to full commercial power generation is expected to begin September 7, 1985.

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NRC TMI PROGRAM OFFICE WEEKLY STATUS REPORT

WEEK ENDING AUGUST 23, 1985

1. PLANT STATUS

- The facility remains in long term cold shutdown with the Reactor Coolant System (RCS) vented to the reactor building atmosphere and the reactor vessel head and plenum assembly removed from the reactor vessel.
- The plenum is on its storage stand in the deep end of the fuel transfer canal. A dam has been installed between the deep and shallow ends of the fuel transfer canal. The deep end is filled with water to a depth of about 20 feet (about 5 feet above the top of the plenum).
- The modified internals indexing fixture is installed on the reactor vessel flange and is flooded to elevation 327 feet 6 inches (15½ feet above the top of the core region).
- Calculated reactor decay heat is less than 12 kilowatts.
- RCS cooling is by natural heat loss to the reactor building ambient atmosphere. Incore thermocouple readings range from 70°F to 92°F with an average of 80°F. Average cold leg temperature is 54°F.
- The average reactor building temperature is 60°F. The reactor building airborne activity is 2.8 E-7 uCi/cc Tritium and 1.6 E-9 uCi/cc particulate, predominantly Cesium 137.

2. WASTE MANAGEMENT

- The Submerged Demineralizer System (SDS) and EPICOR II were shutdown this period.
- Total volume processed through SDS to date is 2,963,375 gallons, and the total volume processed through EPICOR II is 2,547,671 gallons.
- Preparations are being made to transfer the contents of condensate tank 1A (CCT-1A) to EPICOR II. The tank will be desludged and used as a storage tank for borated makeup water.

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ENCLOSURE B

3. DOSE REDUCTION/DECONTAMINATION ACTIVITIES

- Decontamination activities are continuing on the 281' level of the Auxiliary Building.
- Average general area radiation dose rate is 40 mrem per hour on the 347' level of the reactor building and is 67 mrem per hour on the 305' level of the reactor building.

4. ENVIRONMENTAL MONITORING

- EPA sample analysis results show TMI site liquid effluents to be in accordance with regulatory limits, NRC requirements, and the City of Lancaster Agreement.
- TMI water samples taken by the US Environmental Protection Agency at the plant discharge to the river consisted of seven daily composite samples taken from August 3 to August 10, 1985. Gamma scans detected no reactor related radioactivity.
- The Lancaster water sample taken at the water works intake and analyzed by the US Environmental Protection Agency consists of a seven day composite sample taken from August 4 to August 10, 1985. A gamma scan detected no reactor related radioactivity.
- The NRC outdoor airborne particulate sampler at the TMI Site collected a sample between August 15 and August 22, 1985. No reactor related radioactivity was detected. Analysis showed I-131 and Cs-137 concentrations to be less than the lower limits of detectability.

5. REACTOR BUILDING ACTIVITIES

- Installation of the rotating work platform and service platform (over the reactor vessel) has been completed. Work on the north platform is in progress.
- Installation of tool racks for defueling tools continues. The "D" tool rack is nearly complete; work is beginning on the "B" tool rack.
- Defueling Water Cleanup System (DWCS) preoperational testing continued this week. The reactor vessel cleanup portion of the system is expected to be operational by mid-September.

6. AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES

- Installation of the DWCS continued. Partial DWCS turnover for processing RCS during early defueling is expected to be completed in mid-September.
- The second of four fuel canister racks is on site with the remaining two scheduled for delivery in late August.

AUG 23 1985

ENCLOSURE B

7. NRC EVALUATIONS IN PROGRESS

- Technical Specification Change Requests numbers 48, 49, and 50.
- Recovery Operations Plan Change numbers 29, 31, 32, and 34.
- Fuel Canister Technical Evaluation.
- Fuel Handling Senior Reactor Operator Training Program was completed August 22, 1985.
- Defueling Safety Evaluation.
- Application for seismic exemption.
- SDS Technical Evaluation and System Description Update

8. PROJECTED SCHEDULE OF FUTURE EVENTS

- Start of Defueling: October 1985

9. PUBLIC MEETING

The next meeting of the Advisory Panel for the Decontamination of Three Mile Island Unit 2 is scheduled for September 11, 1985. The meeting will be at the Environmental Matters Committee Room (Room 160), House Office Building, College Avenue, Annapolis, Maryland from 6:00 to 9:00 PM. The purpose of the meeting is to inform the public of progress being made in the TMI-2 cleanup and to allow the public to voice concerns and make comments on any aspects of the cleanup. The status of the processed water will also be discussed.

Persons desiring the opportunity to speak before the Panel are asked to contact Mr. Thomas Smithgall at 717-291-1042 or write to him at 2122 Marietta Avenue, Lancaster, Pennsylvania 17603.

OFFICE OF NUCLEAR MATERIAL SAFETY AND SAFEGUARDS

Items of Interest

Week Ending August 23, 1985

Near Term NRC Actions Under the Nuclear Waste Policy Act (NWPA)

Section: 114(e) of NWPA

Action: NRC received DOE Draft Project Decision Schedule on July 18, 1985.

Status: NRC comments are currently being sent to the Commission for concurrence and transmittal to DOE by September 13, 1985.

Section: 301 of NWPA

Action: DOE submitted its Final Mission Plan for the Civilian Radioactive Waste Management Program to Congress and NRC on July 9, 1985.

Status: NRC has been requested to testify on the Mission Plan before the Senate Committee on Energy and Natural Resources on Thursday, September 12, 1985. Proposed testimony is currently being sent to the Commission

Section: 121(a) of NWPA

Action: EPA signed its final high-level waste standards on August 15, 1985.

Status: The standards will be published in the Federal Register the week of August 26-30, 1985. NRC must revise 10 CFR Part 60 to conform to the EPA standards.

NFS-Erwin

The strike by OCAW Union members continues. On August 14, 1985, letters from the Division of Safeguards and the Division of Fuel Cycle and Material Safety, NMSS, were sent to NFS stating that the NRC had no objections to NFS conducting limited operations of the high enriched uranium production and scrap recovery facilities. NFS introduced material into the HEU production area on August 17, 1985.

ENCLOSURE C

AUG 23 1985

General Electric Company, San Jose, CA

License No. SNM-54 was terminated on August 20, 1985. The license, issued in 1967, authorized GE to possess enriched uranium for the production of fuel assemblies for light water reactors. After the start-up of the GE facility in Wilmington, N.C., production was gradually phased out and the major activities at San Jose centered around R&D. The two main buildings where production took place were decontaminated and released for unrestricted use. R&D activities have decreased to the point where these could be assumed under the State license. Accordingly, the State of California amended GE's license to authorize GE to possess small quantities of special nuclear material.

Public Service Electric and Gas Company

On August 21, 1985, Public Service Electric and Gas Company was issued Materials License No. SNM-1953 for Hope Creek Generating Station, authorizing the receipt, possession, inspection, and storage of uranium enriched in the U-235 isotope contained in fuel assemblies in their shipping containers and spent fuel pool.

ENCLOSURE C

AUG 23 1985

OFFICE OF INSPECTION AND ENFORCEMENT
Items of Interest
Week Ending August 23, 1985

1. The following Significant Enforcement Actions were taken during the past week:

- a. EN 85-56, a Notice of Violation and Proposed Imposition of Civil Penalty in the amount of \$5,000 was issued on August 20, 1985 to Astrotech, Inc. (Harrisburg, PA). This action is based on twelve violations which represent a lack of adequate management oversight and control of licensed activities.
- b. EN 85-57, a Notice of Violation and Proposed Imposition of Civil Penalty in the amount of \$100,000 was issued on August 20, 1985 to Florida Power and Light Company (Turkey Point Units 3 & 4). This action is based on the failure of the licensee's staff to determine whether a modification to the spent fuel pits (SFP) piping created an unreviewed safety question. This failure resulted in the operation of SFP systems for several years in a different manner than that described in the Final Safety Analysis Report.
- c. EN 85-58, a Notice of Violation and Proposed Imposition of Civil Penalty in the amount of \$2,500 was issued August 22, 1985 to Hurley Medical Center (Flint, MI). This action is based on multiple violations that represent a breakdown in management oversight and control of the licensee's health physics program. Among the violations involved are: (1) using licensed material in an unauthorized location; (2) improper disposal of licensed materials; and (3) failure to perform surveys.

2. The following IE Preliminary Notifications were issued during the past week:

- a. PNO-I-85-59, Long Island Lighting Company (Shoreham), Unplanned Power Excursion.
- b. PNO-II-85-79A, Nuclear Fuel Services, Incorporated (Erwin, TN), Update on Resumption of Limited Operations Under Strike Conditions.
- c. PNO-II-85-80, Tennessee Valley Authority (Sequoyah Units 1 & 2), Shutdown Greater Than 48 Hours.
- d. PNO-II-85-81, South Carolina Electric & Gas Company (Summer), Unscheduled Shutdown Greater Than 48 Hours.
- e. PNO-II-85-82, Florida Power Corporation (Crystal River Unit 3), High Pressure Turbine Vibration.
- f. PNO-II-85-83, Georgia Power Company (Hatch Units 1 & 2), Receipt of Punctured Waste Oil Drum in Washington State.

ENCLOSURE D

AUG 23 1985

- g. PNO-III-85-71, Commonwealth Edison (Dresden Unit 2), Loss of Offsite Power.
- h. PNO-III-85-71A, Commonwealth Edison Company (Dresden Unit 2), Loss of Offsite Power (Update).
- i. PNO-III-85-72, Commonwealth Edison Company (Byron Unit 2), Low-Level Contamination of Flushing System.
- j. PNO-III-85-73, Commonwealth Edison Company (Byron Units 1 & 2), Offsite Shipment of Contaminated Resins.
- k. PNO-IV-85-39E, Western Stress (Evanston, WY), Apparent Overexposure of Two Radiographers (Final Update).
- l. PNO-IV-85-42, Arkansas Power and Light Company (Arkansas Unit 1), Outage Greater Than Two Days.
- m. PNO-IV-85-43, Nebraska Public Power District (Cooper), Startup After Extended Outage.
- n. PNO-V-85-55, Pacific Gas and Electric (Diablo Canyon Unit 2), Initial Criticality.

3. The following IE Information Notices were issued during the past week:

- a. IE Information Notice 85-71, Containment Integrated Leak Rate Tests was issued August 22, 1985 to all nuclear power reactor facilities holding an operating license or a construction permit.
- b. IE Information Notice 85-72, Uncontrolled Leakage of Reactor Coolant Outside Containment was issued August 22, 1985 to all boiling water reactors holding an operating license or a construction permit.

4. Other Items

a. Senior Management Meeting

Director, IE was in Region II August 22, 1985 to attend the Quarterly Regional meeting with Resident Inspectors.

b. Construction Appraisal Team (CAT) Inspection

Chief, representatives, and consultants to, Reactor Construction Programs Branch, Division of Inspection Programs began CAT inspections at Byron Unit 2 this week. The periods of inspection are scheduled as August 19-30 and September 9-20, 1985.

ENCLOSURE D

AUG 23 1985

c. Small Scale Performance Appraisal Inspections

Representatives of Reactor Operations Branch, Division of Inspection Programs started small scale performance appraisal inspections at D.C. Cook and Fort St. Vrain this week. Inspections will be conducted during the period August 19-23, 1985.

d. Irradiators

Representative of Safeguards Materials Program Branch, Division of Inspection Programs was in Columbus, OH this week to observe the loading of CS 137 into the radiation sterilizer irradiator.

e. Containment Tendons Surveillance

Representative of Engineering and Generic Communications Branch, Division of Emergency Preparedness and Engineering response was at V.C. Summer facility August 21-22, 1985 to review previous containment surveillance reports and to observe vertical tendon anchorages, and at the Vogtle facility on August 23, 1985 to review ongoing tendon installation, greasing procedures, and related information.

f. Transportation of Low-Level Waste

Representative of Safeguards Materials Program Branch, Division of Inspection Programs presented a lecture on transportation of low-level waste at the DOE/ORNL sponsored "Workshop on Research and Development Needs for Treatment of Low-Level Radioactive Waste from Commercial Nuclear Reactor," on August 20, 1985 in Arlington, VA.

g. Vendor Inspections

The Vendor Program Branch, Division of Quality Assurance, Vendor, and Technical Training Center Programs conducted the following inspections this week:

- (1) Nine Mile Point Unit 1 (Syracuse, NY). To review licensee equipment qualification program implementation as required by 10 CFR 50.49.
- (2) Bergen Paterson Pipe Support Corporation (Laconia, NH). To review the cause and corrective action taken on field identified deficiencies in Bergen paterson hydraulic snubbers and to review current quality assurance and manufacturing activities applicable to in-process snubber work.
- (3) Fort Calhoun (Omaha, NE). To participate in an outage inspection with Reactor Construction Programs Branch.

ENCLOSURE D

AUG 23 1985

h. Safeguards Training

Representatives of Operating Reactors Program Branch, Division of Inspection Programs provided safeguards training at the Region II Resident Inspectors meeting on August 22, 1985.

i. Safeguards Meeting

Representative of Operating Reactors Program Branch, Division of Inspection Programs met with the Midwest Nuclear Security Administrators Association this week in Fox Hill, WI to discuss safeguards issues.

j. Emergency Preparedness

A representative of the Emergency Preparedness Branch, Division of Emergency Preparedness and Engineering Response is participating in an inspection of the University of Virginia research reactor this week.

k. Readiness Review Program

On August 19-20, 1985, the Chief, Quality Assurance Branch, Division of Quality Assurance, Vendor, and Technical Training Programs participated in a meeting with Region II and NRR in Atlanta, GA to discuss lessons learned from NRC review of Module One of the Readiness Review Program so as to improve the review process for future modules.

ENCLOSURE D

AUG 23 1985

OFFICE OF NUCLEAR REGULATORY RESEARCH

Items of Interest

Week Ending August 23, 1985

Argonne National Laboratory

A review of the program for modeling of two-phase flow at Argonne National Laboratory (ANL) was conducted on August 14, 1985. The purpose of the ANL program is to assess the scaling compromises in experimental facilities such as the Multi-Loop Integral System Test (MIST) and the University of Maryland (UM) 2 X 4 loop. As such, it is part of the Integral system Test (IST) program that will provide NRR data on the flow regime in the hot leg U-bend which is relevant to post small break loss-of-coolant accident (LOCA) transients for B&W reactors. The hot-leg U-bend experiment is an air-water experimental loop to study two-phase flow regimes, flow separation mechanisms and natural circulation termination for Babcock and Wilcox (B&W) type reactors. Procurement of parts to construct the scaled hot-leg U-bend (4 in. in diameter) facility using Freon is ongoing. Freon is used to simulate the boiling and condensation in the hot-leg U-bend as well as to check fluid property dependency of flow regime transition. The other experiment being conducted is the inverted annular flow experiment using Freon to study the post-critical heat flux (CHF) flow regimes. The inlet to the test section has been modified to give two-phase (slug, churn and bubbly flow) inlet conditions. Data gathering and analysis is expected to be completed in March 1986. The model which is to be developed under this task has the potential of improving the quench front modeling in NRC safety analysis codes such as TRAC and RELAP.

TRAC-BDI/MOD1 Assessment

TRAC-BDI/MOD1 code assessment at the Idaho National Engineering Laboratory (INEL) is now completed. TRAC-BDI/MOD1 is a thermal hydraulics code having capabilities for calculating large and small break LOCAs and system transients in a boiling water reactor (BWR) and has been developed by RES for auditing vendor thermal hydraulic analyses in Chapter 15 submittals or evaluating operating procedures. The code has been assessed by comparing experimental data from various test facilities. The draft NUREG/CR report describing these results has just been received by NRC for comments.

Semiscale

An unsuccessful attempt to run test S-FS-11, a 50 percent secondary feedwater line break was made on August 13, 1985. The test was terminated because of excessive leakage from the primary coolant system. It is postulated, although not known for certain, that a pressure sense line exiting from the top of the long tube in the steam generator was leaking. A similar problem developed earlier in the FS series of tests and was corrected. If the sense line is the cause of leakage, it is estimated that the next attempt to run S-FS-11 will be about August 28, 1985.

Report EGG-SEMI-6985, "Pretest Analysis Document for Test S-FS-11" was also received this week.

Analysis of Potential Severe Core Damage Due to Safeguards-Related Incidents

Results of an analysis of potential severe core damage due to selected deliberate acts at nuclear power reactors were published in a Technical Letter Report, ORNL/NRC/LTR-85-6, July 1985. The purpose of this research project was to explore the feasibility of estimating relative contribution to public risk from known safeguards-related events with an existing analytical technique developed in the Accident Sequence Precursor (ASP) program at ORNL. The study reviewed 11 safeguards-related events that occurred during the period of 1980-1982. In none of these cases was it apparent that a deliberate attempt was made to cause core damage. Using this analytical technique, estimates of potential severe core damage were possible for 5 of the 11 safeguards-related events. The analysis estimated core damage probabilities of values greater than 0.01 in two cases, if additional deliberate acts were assumed. However, the analysts concluded that the likelihood of such additional deliberate acts as a part of each event cannot be estimated because information concerning the intent of the person(s) causing each event is not known. Also, as part of the study, results related to safeguards events were compared to results of previous precursor analyses of safety-related incidents identified from Licensee Event Reports (LERs) in the same time frame.

AUG 23 1985

ENCLOSURE E

Meeting with INEL on Alternatives to Shallow Land Burial

Idaho National Engineering Laboratory (INEL) is under contract to NRC/RES to identify and evaluate the safety features and engineering design assumptions of engineered enhancements and alternatives to shallow land burial (SLB) for low level radioactive waste (LLW) disposal. On August 20, INEL researchers met with RES and NMSS staff to explain their research approach. INEL is applying risk assessment techniques, such as failure mode and effects analysis, to the various engineered components of several SLB alternatives. To help them in their analyses they are using an adaptation of a LLW disposal technology classification scheme which keys on the particular engineered components that comprise various SLB alternatives. This scheme, developed by Rogers and Associates Engineering, Inc. under contract to EPRI, facilitates the study of each engineered feature both as an individual component and as part of an overall engineered system. Moreover, this scheme avoids the potential complication which would arise from considering actual specific alternatives such as the Westinghouse "Sure-Pack" or the French tumulus. Although the work is just beginning, it seems clear that INEL is progressing rapidly and soon will have a useful method for comparing the utility and expected performance of individual components of alternative LLW disposal methods.

Publications to be Issued in the Near Future

Title: Criteria for Power, Instrumentation, and Control Portions of Safety Systems (Regulatory Guide)

Description: This guide describes a method acceptable to the NRC staff for complying with the Commission's regulations with respect to the design, reliability, qualification, and testability of power, instrumentation, and control portions of safety-related systems.

Contact: A. Hintze
443-7712

Title: Criteria for Programmable Digital Computer Systems Software in Safety-Related Systems of Nuclear Power Plants (Regulatory Guide)

Description: This guide describes a method acceptable to the NRC staff for complying with the Commission's regulations for promoting high functional reliability for safety-related systems using programmable digital computers in the operation of nuclear power plants. The method is applicable to designing, verifying, and implementing software, and validating programmable digital computer systems.

Contact: A. Hintze
443-7712

- 4 -

Title: Design of An Independent Spent Fuel Storage Installation (Dry Type)
(Regulatory Guide)

Description: Endorsement of ANSI/ANS 57.9.

Contact: W. Pearson
443-7663

AUG 23 1985

ENCLOSURE E

ITEMS OF INTEREST
OFFICE OF INTERNATIONAL PROGRAMS
WEEK ENDING AUGUST 23, 1985

International Visitor

On Monday Mr. S. Benassai of the Standards Division of Italy's National Committee for Research and Development of Nuclear Energy and Alternative Energy (ENEA) visited NRC for discussions with NRR and RES. Subject areas covered include permissible radiation dosage rates and frequency of in-service inspection of steam generator tubes.

Foreign Trip Report

H. J. Faulkner, Chief, Technical Liaison Section, IP
May 23-June 5, 1985; Visited Taiwan and Japan:

Mr. Faulkner attended the annual meeting of the Joint Standing Committee on Civil Nuclear Cooperation between Taiwan and the U.S. on May 28-29, 1985, in Taipei. Discussions were held with Atomic Energy Council, Taipower, and INER representatives on thermal-hydraulic code assessment, the severe accident program, cooperation in nuclear emergencies, reactor aging and IPIRG programs. On May 30 he visited the Maanshan plant. In Japan Mr. Faulkner met with officials of both MITI and STA to discuss the status of their activities regarding the renewal of our tripartite arrangement on regulatory safety matters and the planned visit to Japan by Chairman Palladino in August.

ENCLOSURE G

AUG 23 1985

OFFICE OF STATE PROGRAMS

ITEMS OF INTEREST

WEEK ENDING AUGUST 23, 1985

Irradiator Workshop

OSP is sponsoring a workshop on radiation safety in design and operation of large pool-type irradiators. State and NRC staff will attend. The workshop will be held 9/4-9/6 at Rutgers University and will include a field trip to a facility recently licensed by NRC.

Meeting with the State of Virginia

On August 21, 1985, OSP and Region II staff members met with the Assistant Health Commissioner and staff in Richmond, Virginia. The State indicated its intention to proceed with full Agreement State status with NRC (excluding regulation of uranium milling) and also discussed its desire to enter into an MOU regarding inspection of low-level radioactive waste packaging and transportation at NRC licensed facilities.

ENCLOSURE H

AUG 23 1985

OFFICE FOR ANALYSIS AND EVALUATION OF OPERATIONAL DATA

ITEMS OF INTEREST

WEEK ENDING AUGUST 23, 1985

On August 21, 1985, AEOD met with the Region II resident inspectors and provided a status report on: AEOD organization and activities, new initiatives, and the proposed Incident Investigation Program. The results of an assessment of the 1984 LERs was also presented. A number of common problem areas and deficiencies noted in these LERs were identified and discussed.

On August 23, 1985, AEOD participated in discussions on the Davis-Besse event with the Region I resident inspectors. AEOD discussed the concept and principles of the proposed Incident Investigation Program and the process to be followed by NRC Teams. This presentation was followed by a discussion of the event by Ernie Rossi, NRC Team Leader, and comments on the investigation and event by Wayne Shafer, Region III Branch Chief.

ENCLOSURE J

AUG 23 1985

OFFICE OF SMALL AND DISADVANTAGED
BUSINESS UTILIZATION/CIVIL RIGHTS

Item of Interest

Week Ending August 23, 1985

Civil Rights Program

On August 15-18, 1985, the Office of Small and Disadvantaged Business Utilization and Civil Rights participated in "Blacks In Government" Seventh Annual National Training Conference, held at the Washington Hilton Hotel. The conference focused on the achievement of merit, equality, and excellence in civil service.

Conference workshops covered such important areas as: career development and management; computer technology and applications; management training and development; financial planning and investments; and performance and productivity improvement.

ENCLOSURE K

AUG 23 1985

DEPUTY EXECUTIVE DIRECTOR FOR
REGIONAL OPERATIONS AND GENERIC REQUIREMENTS
(DEDROGR)

Week Ending August 23, 1985

CRGR

Attached is a summary of the CRGR activities for the period July 1 through July 31, 1985. Background material on topics reviewed by the Committee and the minutes of CRGR meetings are sent to the Commission. Copies are provided to the Public Document Room after the NRC has considered (in a public forum) or decided the matter addressed by the documents. Questions concerning this monthly report should be referred to Walt Schwink (492-8639).

ENCLOSURE M

AUG 23 1985

SUMMARY OF CRGR ACTIVITIES
(July 1 through July 31, 1985)

Meeting No.
Meeting Date
(Announcement
Date)

Agenda

CRGR
Status

CRGR Recommendation
to EDO

EDO Action

79
7/24/85

Review, Proposed Revision 2
of RG 1.99, "Radiation Damage
to Reactor Vessel"

Pending

Additional work needed by both
RES and NRR to address new
information. CRGR recommended
to the EDO that this be given
priority effort. Subsequent to
the meeting, RES requested that
this guide be dropped from
further CRGR consideration
pending coordinated review by
NRR and RES.

Priority EDO action taken
to ensure coordinated
effort by NRR and RES.
Matter brought to
Commission attention by
SECY-85-262 (July 26,
1985 and subsequent memo
of August 22, 1985.

-Sponsoring Office - RES
-Category 2 Item
-Issue #121

Briefing, Request to OMB
for Approval to Continue to
Impose the Reporting & and
Recordkeeping Requirements
of 10 CFR 50

Complete

CRGR strongly encourages staff
to assure that only necessary
reporting and recordkeeping
requirements are imposed by
NRC.

No action necessary.

-Sponsoring Office - NRR
-Category 2 Item
-Item #122

Briefing, Pressure Isolation
Valve Test Requirements

Complete

CRGR suggested staff consider-
ation be given to the develop-
ment of better focused PIV
guidance for the IST reviewers
and to assure plant-specific PIV
requirements for operating
reactors were carried out con-
sistent with NRC backfit procedures
(MC-0514).

No action necessary.

-Sponsoring Office - NRR
-Category 2 Item
-Item #123

ENCLOSURE M

AUG 23 1985

SUMMARY OF CRGR ACTIVITIES
(July 1 through July 31, 1985)

Meeting No. Meeting Date (Announcement Date)	Agenda	CRGR Status	CRGR Recommendation to EDO	EDO Action
79 (Continued)	Review, Proposal for Improving Fire Protection -Sponsoring Office - NRR -Category 2 Item -Issue #117	Complete	CRGR recommended that this issue be forwarded for con- sideration by the Commission after resolution of the con- cerns raised by the CRGR.	No action required at this time because NRR has not yet taken a position on the CRGR recommendation.
78 7/8/85	Review, Proposed Requirements Resulting from Resolution of USI A-46, "Seismic Qualification of Equipment in Operating Plants" -Sponsoring Office - NRR -Category 2 Item -Issue #103	Pending	CRGR recommended issuance subject to review by ELD.	ELD review complete; package approved by EDO for issuance for comment and sent to Commission for information. No further action necessary at this time.
	Briefing, Recommendations on Fire Protection Policy and Program Actions -Sponsoring Office - NRR -Category 2 Item -Issue #117	Pending	Discussion at this briefing was inconclusive. CRGR will review this issue at Meeting #79 (7/24/85).	No action required at this time.

ENCLOSURE M

AUG 23 1985

SUMMARY OF CRGR ACTIVITIES
(July 1 through July 31, 1985)

Meeting No. Meeting Date (Announcement Date)	Agenda	CRGR Status	CRGR Recommendation to EDO	EDO Action
78 (Continued)	Briefing, Proposed New SRP Sections to Address Expansion of Spent Fuel Storage -Sponsoring Office - NRR -Category 2 Item -Issue #118	Complete	CRGR offered several recom- mendations relative to the SRP section, but concluded that its issuance (for public comment), as proposed by NRR, is appropriate.	No action required at this time.
	Briefing, NRC Activities in the Development of NDE Qualification and Performance Demonstration Criteria -Sponsoring Office - RES -Category 2 Item -Issue #120	Complete	The CRGR noted that there appears to be an extensive menu of meaningful technical development work ongoing. Resources should be combined and focused on one path for early and adequate improvement in regulatory requirements.	No action required at this time.
	Review, Proposed Revision to 10 CFR Part 50 of Appendix J (Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors) -Sponsoring Office - RES -Category 2 Item -Issue #111	Complete	Subject to satisfactory staff resolution of CRGR concerns, the CRGR recommends that the EDO forward the proposed Appendix J rule and accompanying RG MS 021-5 to the Commission for approval.	No action necessary at this time because RES has not yet taken a position on the CRGR recommendation.

ENCLOSURE M

AUG 23 1985

Date: August 23, 1985

SUMMARY OF CRGR ACTIVITIES
(July 1 through July 31, 1985)

AUG 23 1985

Meeting No. Meeting Date (Announcement Date)	Agenda	CRGR Status	CRGR Recommendation to EDO	EDO Action
78 (Continued)	Review, Proposed Amendments to 10 CFR 21, Reporting of Defects and Noncompliance and 10 CFR 50.55(e), Reporting of Defects in Design and Construction -Sponsoring Office - IE -Category 2 Item -Issue #119	Complete	CRGR recommends that the proposed amendments be issued for public comment after modifications are made. These modifications should be coordinated with the ROGR staff.	No action required at this time because IE has not yet taken a position on the CRGR recommendations.

ENCLOSURE M

ITEMS ADDRESSED BY THE COMMISSION - WEEK ENDING AUGUST 23, 1985

- A. STAFF REQUIREMENTS - DISCUSSION/POSSIBLE VOTE ON FULL POWER OPERATING LICENSE FOR LIMERICK, 10:30 A.M., THURSDAY, AUGUST 8, 1985, COMMISSIONERS' CONFERENCE ROOM, D.C. OFFICE (OPEN TO PUBLIC ATTENDANCE) Memo SECY to EDO dated 8-21-85

The Commission* met to discuss a full power operating license for Limerick Generating Station, Unit 1.

The Commission heard statements and comments from Phyllis Zitzer, President, Limerick Ecology Action; J.L. Everett, Chairman of the Board and Chief Executive Officer, Philadelphia Electric Company (PECo); and Vincent Boyer, Senior Vice President for Nuclear Power, PECO.

The Commission voted 4-0 to allow the Licensing Board's Fourth Partial Initial Decision to become effective and to authorize the staff to issue a full power license for Limerick Unit 1.

The Commission voted 3-1 (with Commissioner Asselstine disagreeing) to adopt an order (ref: SECY-85-267) implementing the above vote. (This completes Commission action on SECY-85-267.)

(Subsequently, on August 8, 1985 the Secretary signed an order to that effect.)

The Commission further requested the staff to report to the Commission on the general performance of security personnel provided under contract to PECO and other licensees by YOH Security Inc.

(NRB) (RI)

(SECY Suspense: 9/13/85)

* Chairman Palladino was not present.

ENCLOSURE 0

AUG 23 1985

B. SECY 85-220 - ENVIRONMENTAL QUALIFICATION PROGRAM ACTIONS RESULTING FROM
APRIL 2, 1985 COMMISSION MEETING (Memo SECY to EDO dated 8-27-85)

This is to advise you that the Commission (with all Commissioners approving) agreed that the attached Generic Letter, provided to you on August 2, 1985, be sent to all licensees of operating reactors.*

The Commission has also agreed to the following details for reviewing requests for extensions and for imposing civil penalties.

1. The staff shall review any requests for extensions of the November 30, 1985, deadline and provide an analysis and recommendation regarding action to the Commission. Each staff submittal to the Commission should include a recommendation on whether the extension should be granted. The staff recommendation should be sent to the Commission as soon as possible after receipt of the utilities request for extension. The Commission intends to advise licensees well in advance of November 30, 1985, of the Commission's decision on the extension request.
2. Utilities that are unable to comply with 10 CFR 50.49 by November 30, 1985, must have valid, staff approved JCOS to support continued operation. The staff is directed to require licensees to submit current JCOS for each item of non-qualified equipment for staff review and/or approval prior to the licensee being allowed to operate the plant in non-compliance. The Commission wishes to be kept informed of those utilities which express a desire to operate in noncompliance and for each of the utilities in this category, the staff must inform the Commission that the utility has an approved JCO which is valid as of November 30, 1985..
3. The Commission approves the staff's proposal to use its enforcement discretion regarding enforcement actions against the limited number of plants which were not in compliance on or before their deadlines (which were on or before March 31, 1985) and which have submitted untimely requests for extension. In determining whether enforcement is appropriate, the staff will consider whether adequate JCOS were provided and whether an extension would have been granted if timely filed. However, licensees in this situation are subject to the provisions of Generic Letter 85-15 after November 30, 1985.

Attachment: (Not Included)
As Stated

AUG 23 1985

*Generic Letter 85-15 issued 8/6/85

ENCLOSURE 0

AUG 23 1985

NRR MEETING NOTICES*

AUGUST 23, 1985

<u>DATE/TIME</u>	<u>DOCKET NUMBER</u>	<u>LOCATION</u>	<u>PURPOSE</u>	<u>APPLICANT/ ATTENDEES</u>	<u>NRR CONTACT</u>
8/26-29/85	50-498 50-499	Westinghouse Inst. Tech. Trg. Ctr Monroeville, PA	To audit the verification and validation of the QDPS	Houston Lighting & Power Co. Bechtel Westinghouse	N. Kadambi
8/26-30/85 8:00 am	50-245	Millstone Site Waterford, Conn.	To survey administration and organization of the Maintenance and Surveillance Programs	Northeast Nuclear Energy Co.	J. Shea
8/27/85	50-410	Nine Mile Point Site Scriba, NY	To review quality assurance concerning the revetment ditch	Niagara Mohawk Power	M. Haughey
8/27/85 1:30 pm	50-412	P-110 Bethesda	To discuss alternate pipe rupture protection	Duquesne Light Co. Stone & Webster	B. Singh
8/27-28/85 8:30 am		P-422 Bethesda	Working Meeting on specific generic safety issues impacting the EPRI/Industry advanced LWR Requirements Document	EPRI	D. Moran
8/28/85 10:30 am	50-387/ 388	P-110 Bethesda	To discuss DCRDR schedule imple- mentation and unresolved Human Engineering Deficiencies	Pennsylvania Power and Light	M. J. Campagnone

PLEASE BE ADVISED THAT DAILY RECORDING CAN BE HEARD ON 492-7166

*Copies of summaries of these meetings will be made publicly available and placed in the respective docket file(s) in the NRC and local public document rooms.

ENCLOSURE P

AUG 23 1985

NRR MEETING NOTICES*

<u>DATE/TIME</u>	<u>DOCKET NUMBER</u>	<u>LOCATION</u>	<u>PURPOSE</u>	<u>APPLICANT/ ATTENDEES</u>	<u>NRR CONTACT</u>
8/28/85 4:00 pm	50-354	Hope Creek Site Lower Alloways	Safety Parameter Display System Design Verification and Vali- dation Audit	Public Serv. Elec. & Gas Co.	D. Wagner
8/28/85 1:00 pm	50-286	P-114 Bethesda	To discuss exemptions to Appendix R	Power Authority of the State of NY	J. Neighbors
8/28/85 8:00 am	50-346	Davis Besse Site Oak Harbor, Ohio	Discuss status of licensee's trouble-shooting and correc- tive actions, plant visit and discuss planned actions	Toledo Edison	A. De Agazio
8/29/85 9:00 am	50-528,529,530	MNBB-6110 Bethesda	To discuss scope of the small break LOCA analysis	Arizona Public Serv. (Palo Verde)	M. Ley
9/4/85 9:00 am	50-346	P-422 Bethesda	Discuss licensee's planned actions relative to 6/9 event	Toledo Edison Co.	A. DeAgazio
9/4-5/85	50-289	P-114 Bethesda	Resolve open issues regarding Inservice Testing Program for pumps and valves, TMI-1	GPU Nuclear Corp.	
9/4-6/85 9:00 am	50-443/444	Seabrook Site Seabrook, NH	Assess status of construction and completion schedule	Public Service Co. of New Hampshire	V. Nerses

PLEASE BE ADVISED THAT DAILY RECORDING CAN BE HEARD ON 492-7166

*Copies of summaries of these meetings will be made publicly available and placed in the respective docket file(s) in the NRC and local public document rooms.

ENCLOSURE P

NMSS MEETING NOTICES

FOR WEEK ENDING: 08/23/85

Division of Fuel Cycle and Material Safety

<u>DATE/TIME</u>	<u>DOCKET NUMBER</u>	<u>LOCATION</u>	<u>PURPOSE</u>	<u>ATTENDEES/ APPLICANT</u>	<u>NRC CONTACT</u>
8/26-30/85		Albuquerque, NM	To attend workshop on subcritical measurements.	G. H. Bidinger, FC	Bidinger
9/3/85		DOT, Washington, D.C.	Pre-application meeting with DOT for revalidation of Castor MTR Cask.	C. E. MacDonald, FC C. R. Chappell, FC R. R. Rawl, DOT GNS Representatives	MacDonald
9/3-6/85		New Brunswick, NJ	Attend large irradiator radiation safety workshop for regulatory personnel.	D. R. Chapell, FC N. Bassin, FC J. Hickey, FC B. Carrico, FC	Bassin
9/5/85		DOE, Washington, D.C.	Meeting to discuss DOE/NRC/DOT Regulatory/certification issues.	R. E. Cunningham, FC C. E. MacDonald, FC J. Cook, FC L. Barrett, et al, DOE Contractors	Cook
9/5-10/85		Idaho Falls, Idaho	To discuss special site criteria in EG&G developed reports.	W. A. Nixon, FC	Nixon
9/6/85 (postponed fm. earlier date)		5th. Floor Conf. Room, SS	Meeting to discuss nuclear stethoscope training requirements.	R. E. Cunningham, FC V. Miller, FC N. McElroy, FC W. J. Walker (Health Physics Svs., Inc.)	Cunningham
9/6/85		Oak Ridge, TN	Participate in Tennessee city/county task force on monitored retrievable storage.	C. E. MacDonald, FC DOT - TN State and local ofcls.	MacDonald
9/9-12/85		Jackson, WY	To attend and participate in 1985 Nuclear Criticality Safety Topical Meeting.	N. Ketzlach, FC G. H. Bidinger, FC K. Kodali, FC	Ketzlach

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NMSS MEETING NOTICES

FOR WEEK ENDING: 8/23/85

Division of Fuel Cycle and Material Safety

<u>DATE/TIME</u>	<u>DOCKET NUMBER</u>	<u>LOCATION</u>	<u>PURPOSE</u>	<u>ATTENDEES/ APPLICANT</u>	<u>NRC CONTACT</u>
9/10/85		11th Floor Conf. Room, Willste Bldg., SS	Meeting with United Kingdom officials regarding 10 CFR Part 20, exemption levels in <u>de</u> <u>minimis</u> , and high-level and low-level waste.	R. Clark, UK G. Webb, UK R. E. Cunningham, FC D. A. Cool, FC Representatives of WM, RES, and NRR	Cunningham

Division of Waste Management

None

Division of Safeguards

None

ENCLOSURE P

AUG 23 1985

RES MEETING NOTICES

August 23, 1985

<u>DATE/TIME</u>	<u>DOCKET NUMBER</u>	<u>LOCATION</u>	<u>PURPOSE</u>	<u>ATTENDEES/ APPLICANT</u>	<u>NRC CONTACT</u>
9/5/85 1:30 PM		AR	Review status & draft report for FIN B8973 on task analysis of control room crews application: Part II Skills, Knowledge & Training	GPC NRR RES	Au
9/5-6/85		NL	Review handbook on statistical methods for nuclear material accounting	PNL RES RM	Frattali
9/19/85 9:00 AM- 5:00 PM		Auditorium, National Academy of Sciences Building 2101 Constitution Ave., NW	Seminar on Liquefaction of Soil During Earthquakes Hazard at Saturated Soil Sites: Evaluation and Mitigation		Beratan
10/22-25/85		NBS Gaithersburg, MD	13th Water Reactor Safety Research Information Meeting (Preliminary Agenda attached)		Burda

ENCLOSURE P

13th Water Reactor Safety Research Information Meeting

This is a reminder that the Office of Nuclear Regulatory Research will hold its Thirteenth Water Reactor Safety Information Meeting at the National Bureau of Standards on October 22-25, 1985. The 4-day meeting will cover the results and plans of NRC's nuclear safety research programs.

Hundreds of representatives of government, industry, universities, public interest research groups and foreign nations have been invited to the 13th Water Reactor Safety Research Information Meeting.

The 4-day session will focus on: Integral Systems Tests, Severe Accident Source Term, Seismic Research, Risk Analysis, Materials Engineering Research, Mechanical and Structural Research, Containment Systems Research, and the International Code Assessment Program.

Enclosed are copies of the preliminary agenda. Please give them wide distribution within your organization. Please note that NRC employees are not required to register unless they wish to have lunches provided and coffee breaks. The registration fee is \$15.00 per day.

Further information about the meeting and copies of the preliminary agenda can be obtained by calling Al Burda, Office of Nuclear Regulatory Research, 427-4468.

AUG 23 1985

ENCLOSURE P

THE DAY MORNING - OCTOBER 22, 1985

Preliminary Agenda, July 22, 1985

RED AUDITORIUM

GREEN AUDITORIUM

LECTURE ROOM B

Plenary Session

Chairperson: G. Naras (NRC)

9:15 Introduction: G. Naras (NRC)

9:20

- Speakers to be announced in final agenda
-
-

10:20 Break

Integral Systems Tests

Chairman: W. D. Becker (NRC)

10:40 FIRST Analysis, R. Sutherland (GE)

11:10 0715 Test Results, J. Gloumanis (NRC)

11:40 Delivery of Newland Test Facility Results,

Y. Y. Mao (GE, M&J)

12:10 WEST Facility Status, R. Carter (NRC)

12:40 Discussion

1:00 Lunch

Risk Analysis/PRA Applications

Chairman: G. Burdick (NRC)

10:40 Operational Phase of Inspection Prioritization,

D. Campbell (BET Associates)

11:05 The Use of Risk Analysis in Evaluating Technical

Specifications, J. Bocco (NRC)

11:35 PRA Application in Operation, J. H. Bichel

(Northwest Utilities)

12:00 Use of PRA in Regulatory Consideration of Severe

Accidents, Z. Roztoczy and T. Speis (NRC)

12:30 Risk Analysis of Decay Heat Removal Sequences During

Shutdown, W. Reuland and J. Gaerthner et al. (EPRI)

1:00 Lunch

Process Control

Chairman: K. G. Sayer (NRC)

10:40 Facilitation of Decommissioning LWRs, T. LaGuardia

(TEG Engineering)

11:10 Decommissioning Impacts on Solidification and Waste

Disposal, P. Piculfo (NRC)

11:40 Decommissioning: On-site Experience, B. Brunano

(NRC Nuclear Industries)

12:10 LWR Spent Fuel Rod Behavior During Long-Term Dry Fuel

Storage Conditions, C. S. Olson (EG&G)

12:40 Lessons Learned from a NPP-073: Review of High-Range

Effluent Monitors and Samplers, A. P. Hall (NRC) and

J. R. White (NRC)

1:00 Lunch

Integral Systems Tests

Chairman: D. E. Solberg (NRC)

2:00 Introduction, D. E. Solberg (NRC)

2:15 Simulacra Feedwater Line/Steam Line Break Test Results,

T. Bouchier (EG&G)

2:45 Simulacra Liquid Holdup Test Results and Comparison

with S-07 Results, T. Bouchier (EG&G)

3:15 Break

3:30 ROSA II Test Results, R. Tiesdel (JAEI)

3:35 ml. Reflood Tests Including End-of-Blooddown, B. Brand,

R. Heidl and H. Weitzinger (NRC)

4:20 Test Reporting in Support of Computer Code Assessment

and Uncertainty Quantification, R. Shaw (EG&G)

4:45 IST Scaling Study, T. Larson (EG&G)

5:10 Continuing Integral Testing Capability - Approach and

Scaling Study Results, T. Larson (EG&G)

5:35 Adjourn

Mechanical and Structural Research

Chairman: J. J. Burns (NRC)

2:00 NRC Overview of Mechanical/Structural Research,

J. Richardson (NRC)

2:15 Structural Load Combinations, H. Hwang (NRC)

2:35 Standard Problems for Structural Computer Codes,

M. Reich (NRC)

2:55 Containment Buckling Research Results, J. Bennett

(LANL)

3:15 Break

3:30 Piping Overview, D. Gatz (NRC)

3:50 Pipe Damping, J. Weir (NRC)

4:10 Piping Capacity Tests, A. Ouesha (ETEC)

4:30 Pipe Ruptures in BWR Reactors, G. Holman (LLNL)

4:50 Valve Performance Testing, M. Jeannoulin (ETEC)

5:15 Adjourn

THE DAY AFTERNOON - OCTOBER 22, 1985

Nuclear Plant Aging

Chairman: J. P. Vora (NRC)

2:00 An Approach to Understand Aging - System Interactions

and Evaluation, J. Cleveland (SEA, Inc.)

2:40 An Update on SQUG Activities, A. Marion (EG&G)

3:15 Break

3:30 Materials Aspects of BWR Plant Life Extension,

B. M. Gordon (GE)

4:05 The Measure of Equipment Degradation, S. Carlingno and

G. Tamba (EPRI)

4:40 An Approach to Evaluate Safety and Risk Implications of

Aging and Service Wear, M. Vesley (NRC)

5:15 Adjourn

MEET ADDITION

Severe Accident Sequence Analysis
Chairman: R. T. Curtis (NRC)

- 9:15 Welcoming Statement, R. T. Curtis (NRC)
- 9:20 Application of RAISON to BWR ATWS, P. Saha and G. Slovits (BNL)
- 9:40 TRACE-MELCOR Analysis of TMLB Transients in Occurrence-1, B. E. Boyack and T. J. Henninger (LANL)
- 10:00 ATWS Analysis for Brown's Ferry Nuclear Plant Unit 1, J. Bellman and W. Jones (INEL)
- 10:20 Break
- 10:35 RELAP/MC202 Ballistic Analysis of Feedwater Transient Initiated Sequences, C. Brodie (INEL)
- 10:55 The UNREA MWR Severe Accident Containment Study, A. T. D. Bullard, B. D. Turland and R. L. D. Young (ORNL)
- 11:20 Analysis of Brown's Ferry Station Blackout Transient, L. J. Orr, C. R. Hyman and C. F. Weber (EPRI)
- 11:50 Comparison of Peach Bottom Station Blackout Calculations (ORNL/BNL), S. A. Hodge (ORNL)
- 12:05 MELCOR - Development and Use, A. Sauer (ORNL)
- 12:20 Hydrogen Transport in a Large Dry Containment for Selected Arrested Sequences, D. B. King and A. C. Peifer (SMI)
- 12:40 Pressure/Temperature Response in an Ice Condenser Dismantling for Selected Accidents, S. E. Dillingham et al (SNL)
- 1:00 Lunch

Fission Product Release & Transport in Containment
Chairman: T. J. Walker (NRC)

- 2:00 Status of the ORNL Aerosol Release and Transport Project, R. E. Adams (ORNL)
- 2:20 Status of the DEMON Experiments - A Comparison with NAUA Calculations, L. Benz et al. (KTH-FRC)
- 2:40 Status of the LWR Aerosol Containment Experiments (LANL) Program, G. R. Bloom et al. (NRC)
- 3:00 Containment Integrity Under Severe Accident Conditions, W. A. von Rosenberg et al. (SNL)
- 3:20 Break
- 3:35 Experimental Validation and Improvement of Core Debris/Concrete Interaction Models, J. Grolinger, D. Bradley, A. San-Martín, J. Brocman and E. Copus (SMI)
- 3:55 Review of the Large-Scale Core-Concrete Interaction Experiments and Analysis at the KTH Beta Facility, FRG, R. K. Oble, Jr. and W. Reimann (KTH)
- 4:15 Pressurized Ejection of Molten Core Debris and Direct Containment Heating, W. M. Terbell, M. Plich and J. E. Brocman (SNL)
- 4:35 Analysis of Molten Fuel-Concrete Interactions and Fission Product Release from Ex-Vessel Core Debris, D. R. Bradley and D. A. Powers (SNL)
- 4:55 BNL Severe Accident Sequence Experiments and Analysis Program, G. A. Greene, T. Ginsberg and M. Tufu (BNL)
- 5:15 Adjourn

LECTURE ROOM B

Seismic Research
Chairman: J. J. Burns (NRC)

- 9:15 NRC Overview of Seismic Research, J. Richardson (NRC)
- 9:35 Eastern U.S. Seismic Hazards, R. Murphy (NRC)
- 9:55 Seismic Design Margins, R. Budnitz (PRA)
- 10:15 Break
- 10:30 Seismic Category I Structures Program, J. Bennett (LANL)
- 10:50 Component Fragilities - Data Collection, Analysis and Interpretation, C. Holmeyer (NRC)
- 11:10 Component Fragilities - Testing Grouping and Priorities, G. Holman (LANL)
- 11:30 Cooperative Efforts for Vetting of Seismic Calculations, J. Cosulich (NRC)
- 11:50 Design of a 2 Kiloton Eccentric Mass Vibrator and HDR Test Matrix, L. Melcher (KTH)
- 12:10 SSGP - BWR Risk Assessment, G. Cummings (LLNL)
- 12:30 Discussion
- 1:00 Lunch

Equipment Qualification
Chairman: W. S. Farmer (NRC)

- 2:00 Status of Japanese Equipment Qualification Research, S. Ohada (JAERI)
- 2:25 Equipment Qualification and Survival Research at SNL, L. Bonzon (SNL)
- 2:50 Fire Protection and Hydrogen Burn Equipment Survival Research, D. Barry (SNL)
- 3:20 Break
- 3:35 Environmental and Dynamic Qualification of Mechanical Equipment Research at INEL, J. Hunter (INEL)
- 4:20 PMEL: Major Outstanding Equipment Survival/Qualification Issues, L. Bonzon, D. Barry, J. Hunter and J. Rhoads (EPRI)
- 5:20 Adjourn

WEDNESDAY AFTERNOON - OCTOBER 23, 1985

International Code Assessment Program
Chairman: F. Odier (NRC)

- 2:00 Methodology for Code Accuracy Quantification, L. Burton (SNL)
- 2:20 Uncertainty Development and Application, G. Wilson (EG&G)
- 2:40 Improvements of BWR LOCA/ECCS Analysis in Japan, K. Yabagi (TEPCO)
- 3:00 New Japanese Correlation on Core Cooling and CCFL Characteristics During LOCA, H. Nagasaka (Tohoku)
- 3:20 Break
- 3:35 Application of TBL Results to BWR Plants, W. Helton (Hatch)
- 3:55 SAFER-03 Qualification by TBL Test Analysis, T. Sugisaki (Hitachi)
- 4:15 SAFER-03 Qualification Against ROSA-III Recirculation Line Break Spectrum Tests, K. Itoye (WIG)
- 4:35 Assessment of SAFER Code for LOCA with Advanced Boiling Water Reactor Test Facility, B. Shirahar (GE)
- 4:55 Safety Performance of Advanced Boiling Water Reactor, C. Sawyer (GE)
- 5:10 Adjourn

RED AUDITORIUMSevere Accident Source Term

Chairman: M. Silberberg (NRC)

- 9:15 Welcoming Statement, M. Silberberg (NRC)
 9:20 Review of Source Term Reassessment, M. Silberberg (NRC)
 9:35 Source Term Code Package, J. Giesseke and P. Cybulskis (BCL) and R. Berl and T. Prell (BNL)
 10:15 Break
 10:30 Severe Accident Code Development Program, J. Han (NRC)
 10:45 RELAP-SCOMP-TRANMELT-PARAGRASS Computer Code, G. Allison (INEL)
 11:10 TRAC-MELPROG Computer Code, J. Kelly (SML)
 11:30 Discussion
 11:40 Iodine Release During a Hydrogen Burn, L. Nelson (SML)
 12:05 Recent Evaluations of Tellurium Behavior During the TMI-2 Accident, (INEL)
 12:40 Aerosol Retention in Ice Condenser Containments, P. Deczaraki (PNL)
 1:00 Lunch

GREEN AUDITORIUMInternational Code Assessment Program

Chairman: D. E. Bessette (NRC)

- 9:15 Application of RELAP5 to Analysis of Dual Steam Generator Tube Rupture and Studies of Loss of Feedwater Line Break Transients, E. Stubbs (Belgium)
 9:30 F.K. Germany Assessment of TRAC-PF1/MOD1 and RELAP5/MOD2 Being Conducted by KRI, F. Winkler (FRG)
 10:20 Break
 10:35 Technical Research Center of Finland Use of RELAP5/MOD2, M. Holmstrom (Finland)
 11:10 Assessment and Application of RELAP5/MOD2 at Studsvik; Use of TRAC-PF1/MOD1 to Analyze Loss of Grid Transient, O. Sanderberg (Sweden)
 11:45 Analysis of BWR6 LOCA with Reduced ECCS Using TRAC-BD1/MOD1, S. N. Aksun (Selferland)
 12:20 United Kingdom Experience Gained with TRAC-PF1/MOD1 and RELAP5/MOD2, J. Fell (UK)
 1:00 Lunch

LECTURE ROOM BSurry Steam Generator/
Examination and Evaluation
Chairman: J. Muscare (NRC)

- 9:15 Generator Degradation Characterization, R. A. Clark (PNL)
 10:10 Break
 10:25 Decontamination and Cleaning, R. A. Clark (PNL)
 11:15 Eddy Current Inspection Round Robin, P. G. Doctor and R. Ferris (PNL)
 12:30 Steam Generator Tube Vibration Study, W. L. Enderlin (PNL)
 12:45 Stress Corrosion Cracking of Steam Generator Tubing, D. van Rooyen (BNL)
 1:00 Lunch

THURSDAY AFTERNOON - OCTOBER 24, 1985

Risk Analysis/
Dependent Failure Analysis

Chairman: D. Rasmussen (NRC)

- 2:00 An Overview of the Risk Methods, Integration, and Evaluation Program, J. C. Shepherd (NRC)
 2:15 Analysis of Dependent Failures and External Events, M. P. Bohn (SML)
 2:45 An NRC Approach to Dependent Failure Analysis, D. Campbell (JBF Associates)
 3:15 Break
 3:30 An Industry Approach to Dependent Failure Analysis, K. Fleming (PLG)
 4:00 Integrating Root Causes into PRA's, W. E. Vesely (BCL)
 S. Bruske and L. Cadwallader (INEL)
 4:30 Root Cause Analysis of Component Data, G. Orellin and A. Seith (LATA)
 5:00 Discussion
 5:35 Adjourn

Materials Engineering Research
Non-Destructive Evaluation

Chairman: J. Muscare (NRC)

- 2:00 Primary Systems Integrity Research Overview, C. Z. Serpan (NRC)
 2:05 Nondestructive Examination Reliability, S. R. Doctor (PNL)
 2:35 Field Implementation of SAFT-UT, S. R. Doctor (PNL)
 3:00 Leak Detection and Nondestructive Examination of Stainless Steel, D. S. Kupperman (ANL)
 3:20 Break
 3:35 Progress in Real-Time, on-Line Acoustic Emission Monitoring of Cracks in Reactor Systems, P. H. Hutton (PNL)
 3:50 Improved Eddy Current Procedures, C. V. Dodd (ORNL)

Environmental Effects in Piping
Chairman: A. Taboada (NRC)

- 4:05 Effects of Welding and Weld Repair on Stainless Steel Piping, S. Rummel (PNL)
 4:25 BWR Pipe Cracking and Weld Clad Overlay Studies, W. J. Shack (ANL)
 4:55 Aging of Cast Stainless Steels, O. K. Chopra (ANL)
 5:35 Adjourn

Containment Systems Research
Containment Loads Analysis
Chairman: P. M. Wood (NRC)

- 2:00 Models for BWR and PWR Engineered Safety Features in the CONTAIN Code, F. J. Schelling, R. K. Murata, D. C. Williams and P. E. Rucroth (SML)
 2:20 Validation, Assessment and Applications of the CONTAIN Computer Code, K. D. Bergeron, R. K. Murata, D. C. Williams and J. L. Tills (SML)
 2:40 Molten Core-Coolant Interaction Analysis Using SIMMER-II, M. R. Bohl (LANL)
 3:00 Short Term and Long Term Aspects of Accretion Containment Tests, L. Wolf, L. Valencia and K. H. Scholl (KTH-FRG)
 3:20 Break
 3:35 HWS-BURN: A Model for Hydrogen Distribution and Combustion in Nuclear Reactor Containments, J. R. Travis (LANL)
 3:55 HECTR Development and Assessment, C. C. Wong (SML)
 4:15 Quench-Spray Modeling of Flame Acceleration and Air Flow, K. D. Marx (SML)
 4:35 FLAME Acceleration and Detonation: Research, M. P. Sherman and S. R. Tiesler (SML)
 4:55 Platinum Catalytic Igniters for Lean Hydrogen-Air Mixtures, L. R. Thorne, J. V. Voiponi and W. M. McLean (SML)
 5:15 The Effect of Droplet Size on the Calculation of Pressure Difference in Subcompartment Analysis, K. Almones (U-M)
 and R. Lee (NRC)
 5:25 Adjourn

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FRIDAY MORNING - OCTOBER 25, 1985

RED AUDITORIUM

Materials Engineering Research

Pressure Vessel Research

Chairman: M. Vagins (NRC)

- 9:15 Crack Arrest Studies Overview, C. Pugh (ORNL)
- 9:25 Wide Plate Experiments, R. Fields (NBS)
- 9:45 Dynamic Fracture Analyses, R. Bass (ORNL)
- 10:00 Development of Viscoplastic Fracture Mechanics,
M. Kanninen (SuRI)
- 10:20 Break
- 10:35 Final Results PTSE-1; Plants for PTSE-2,
C. Pugh (ORNL)
- 10:55 Reactor Vessel Cladding; Separate Effects Study,
W. R. Corwin (ORNL)
- 11:20 Irradiation Effects on Reactor Vessels, Steels and
Weldment (HSST 4th & 5th Series), R. K. Hanstad
(ORNL)
- 11:45 Radiation Sensitivity and Annealing Parameter
Studies, J. R. Hawthorne (NEA)
- 12:15 LWR-Dosimetry Program Overview, W. N. McElroy (HEDL)
- 12:40 Reactor Vessel Cavity Dosimetry, J. Butler (UKAEA)
- 1:00 Lunch

GREEN AUDITORIUM

Code Assessment and Improvement

Chairman: F. Oder (NRC)

- 9:15 TRAC PFI/MOD1, D. Liles (LANL)
- 9:35 TRAC-BWR, Z. Rouhani (EG&G)
- 9:55 RELAPS, G. Johnsen (EG&G)
- 10:15 Break
- 10:30 COBRA/TRAC, D. Trent (PHL)
- 10:50 RAMONA, P. Sahe (BNL)
- 11:10 Overview of TRAC-BWR Assessment;
TRAC-BDI/MOD1-TRAC-BFI, G. Wilson (EG&G)
- 11:30 LANL TRAC-PFI/MOD1 Code Assessment Program Using
LOFT and OTIS Data, T. Knight (LANL)
- 11:50 Assessment of TRAC-BDI/MOD1 Using FIST Data,
P. Sahe (BNL)
- 12:10 RELAPS/MOD2 Assessment at INEL, P. Wheatley (INEL)
- 12:30 Status of TRAC-PFI/MOD1 Independent Assessment at
Sandia, L. Kmetz (SNL)
- 1:00 Lunch

LECTURE ROOM B

Industry Safety Research

Chairman: W. B. Loewenstein (EPRI)

- 9:15 Safety Research in Transition, W. B. Loewenstein et al.
(EPRI)
- 9:45 Steam Generator Tube Integrity, J. Lang and P. Kalra
(EPRI)
- 10:15 Break
- 10:30 Signal Validation: A New Industry Tool, B. Sun and
M. Divekaruni (EPRI)
- 11:10 On-Line Corrosion Cracking Monitor Development, J. Gilman
and R. Jones (EPRI)
- 11:50 Severe Accident Containment Integrity, H. Tang et al.
(EPRI)
- 12:25 Post APS Source Term Research, R. Vogel et al. (EPRI)
- 1:00 Lunch

FRIDAY AFTERNOON - OCTOBER 25, 1985

Materials Engineering Research
Piping Research and Fracture Mechanics

Chairman: M. Mayfield (NRC)

- 2:00 UK PWR Materials and NDE Research, G. J. Lloyd (UKAEA)
- 2:20 NRC Piping Review Committee; Summary on Pipe Breaks,
R. Klecker (NRC)
- 2:35 Degraded Piping Program - Phase II Progress,
G. Wilkowski (BCL)
- 3:00 Break
- 3:15 Piping Research in the Federal Republic of Germany,
M. Schulz (GRS)
- 3:40 Piping Research in Italy, P. Milella (ENEA)
- 4:05 Piping Research in Japan, G. Yagawa (U. Tokyo)
- 4:30 Fracture Experiments on Welded Stainless Steel Pipe,
R. Hayes (OTHSRDC)
- 4:50 Piping Fracture Mechanics Data Base, A. L. Hiser (NEA)
- 5:10 Adjourn

20/30 Research

Chairman: G. S. Rhee (NRC)

- 2:00 Results of CCTF Tests, Y. Murgo (JAERI)
- 2:35 Results of SCTF Tests, H. Adachi (JAERI)
- 3:15 Break
- 3:30 Analytical Support for CCTF, SCTF and UPFT Tests,
J. Spore (LANL)
- 4:00 Status of German UPTF Program, K. R. Hofmann (GRS)
- 4:30 Adjourn

Nuclear Plant Analyzer

Chairman: C. R. Troutman (NRC)

- 2:00 INEL Plant Analyzer, E. T. Laats (EG&G)
- 2:35 LANL Plant Analyzer, D. Liles (LANL)
- 3:15 Break
- 3:30 BNL Development of BWR Plant Analyzer, W. Muller,
H. S. Cheng, R. H. Mallen and E. Caszili (BNL)
- 4:00 Adjourn

ENCLOSURE P

AUG 23 1985

August 23, 1985

RII MEETING NOTICE

<u>DATE/TIME</u>	<u>DOCKET NUMBER</u>	<u>LOCATION</u>	<u>PURPOSE</u>	<u>ATTENDEES/ APPLICANT</u>	<u>NRC CONTACT</u>
08/29/85 7:30 pm		Soddy-Daisy, Tennessee	Presentation of Operator License at Sequoyah	Licensee Operators, TVA Management rep., Regional Administrator, selected Regional and Resident Staff Members	Grace

ENCLOSURE P