

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Callaway Plant Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 4 8 3 1				PAGE (3) OF 0 2			
TITLE (4) Reactor Trip Due to Inadvertent MFIV Closure																	
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER (5)				
0 7	3 1	8 5	8 5	0 3	6	0 0	0 8	2 8	8 5					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)															
1		20 402(b)				20 405(c)				<input checked="" type="checkbox"/> 50 73(a)(2)(iv)		73.71(b)					
POWER LEVEL (10)		20 405(a)(1)(i)				50 36(e)(1)				50 73(a)(2)(iv)		73.71(c)					
1 0 0		20 405(a)(1)(ii)				50 36(e)(2)				50 73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 365A)					
		20 405(a)(1)(iii)				50 73(a)(2)(ii)				50 73(a)(2)(viii)(A)							
		20 405(a)(1)(iv)				50 73(a)(2)(iii)				50 73(a)(2)(viii)(B)							
		20 405(a)(1)(v)				50 73(a)(2)(iv)				50 73(a)(2)(ix)							
LICENSEE CONTACT FOR THIS LER (12)																	
NAME Michael E. Taylor - Superintendent, Operations										TELEPHONE NUMBER AREA CODE 3 1 4 6 7 6 - 8 2 0 7							
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																	
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC							
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR	
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO							

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1444 CDT on 7/31/85, a Reactor Trip occurred as a result of a low steam generator (S/G) level when a main feedwater isolation valve (MFIV) was inadvertently closed. A Feedwater Isolation, Auxiliary Feedwater Actuation, and S/G Blowdown Isolation occurred per design and the required safety equipment performed as expected. At the time of the trip the reactor was in Mode 1, Power Operation, at 100% power and normal operating temperature and pressure.

The MFIV closed when an operator inadvertently depressed the control switch for the valve. The switch is located at the front edge of the control panel and was accidentally depressed by the operator. The operators attempted to maintain S/G level however the trip occurred. The operators recovered from the trip and stabilized plant conditions per plant operating procedures.

To prevent recurrence, "stop and think" protective covers have been installed on the valve control switch and on similar switches on the control panel.

A loss of feedwater flow is an analyzed accident condition for which mitigation is provided by safety features included in the plant design. The appropriate safety features responded per design and thereby prevented this incident from endangering the public health and safety.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)

Callaway Plant Unit 1

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

YEAR SEQUENTIAL REVISION
NUMBER NUMBER NUMBER

0 5 0 0 0 4 8 3 8 5 - 0 3 6 - 0 0 0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1444 CDT on 7/31/85, a Reactor Trip occurred as a result of a low level in steam generator (S/G) "A." The low S/G level occurred when the associated main feedwater isolation valve (MFIV), AE-FV-0039, ⁽¹⁾ was inadvertently closed. An Auxiliary Feedwater Actuation and S/G Blowdown Isolation were also initiated by the low S/G level and a Feedwater Isolation resulted from the Reactor Trip. The required safety equipment performed as designed. At the time of the event the plant was in Mode 1, Power Operation, at 100% power and normal operating temperature (588°F.) and pressure (2235 p.s.i.).

The MFIV slow-closed when a licensed operator inadvertently depressed the control switch for the valve, AE-HIS-0039. ⁽²⁾ The switch is located near the front edge of the control panel and was accidentally depressed by the operator while he was reviewing the status of plant equipment during a shift turnover. The closing of the MFIV isolated feedwater flow to S/G "A" and level in the S/G began to decrease.

In an attempt to maintain the S/G level, the operators reduced load. S/G "A" level fell below the low-low S/G level Reactor Trip setpoint of 23.5% and the trip occurred. The operators recovered from the trip and stabilized plant conditions per plant operating procedures. Recovery procedures were completed at 1545 on 7/31/85.

To prevent recurrence, "stop and think" protective covers were installed on valve control switch AE-HIS-0039 and on nine other switches which are in similar locations on the control panel. The covers were installed on 7/31/85. The protective covers had not been installed prior to this incident because sufficient protection against inadvertent actuations of these switches was believed to be provided by a guard rail on the front edge of the control panel. Additional protective covers were placed on those switches which had the potential for inadvertently degrading plant conditions and subsequently actuating the Reactor Protection System or Engineered Safety Features.

A loss of feedwater flow is an analyzed accident condition for which mitigation is provided by safety features included in the plant design. The appropriate safety features responded per design and thereby prevented this incident from endangering the public health and safety.

Previous occurrences: none

(1) MFIV: IEEE Standard 805-1983 System - SJ
IEEE Standard 803A-1983 Component - ISV

(2) Control Switch: IEEE Standard 805-1983 System - SJ
IEEE Standard 803A-1983 Component - HIS

UNION ELECTRIC COMPANY
CALLAWAY PLANT

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August 28, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-1164

Gentlemen:

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 85-036-00
REACTOR TRIP DUE TO INADVERTENT MFIV CLOSURE

The enclosed Licensee Event Report is submitted pursuant to
10 CFR 50.73(a)(2)(iv) concerning a Reactor Trip and Engineered Safety
Features Actuations which resulted from an inadvertent closure of a
Main Feedwater Isolation Valve.

S. E. Miltenberger
for S. E. Miltenberger
Manager, Callaway Plant

met/jwk
MET/jwk/drs
Enclosure

cc: Distribution attached

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cc distribution for ULNRC-1164

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