

Commonwealth Edison Company
Dresden Generating Station
6500 North Dresden Road
Morris, IL 60450
Tel 815-942-2920



February 10, 1997

JSPLTR 97-0026

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

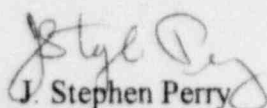
Subject: Monthly Operating Data Report for January 1997
Dresden Nuclear Power Station
Commonwealth Edison Company
Docket Nos. 50-010, 50-237, and 50-249

Gentlemen:

Enclosed is the Dresden Nuclear Power Station Monthly Operating Summary Report for January 1997.

This information is supplied to your office as required by Technical Specification 6.9.A.5, in accordance with the instructions set forth in Regulatory Guide 1.16.

Sincerely,


J. Stephen Perry
Site Vice President
Dresden Station

Enclosure

cc: NRC Region III Office
Illinois Dept. of Nuclear Safety, State of Illinois
NRC Senior Resident Inspector
UDI, Inc. - Washington, DC
File/Numerical

IE24/1

190007

9702190228 970131
PDR ADOCK 05000010
R PDR

MONTHLY NRC

SUMMARY OF OPERATING EXPERIENCE,

PER REGULATORY GUIDE 1.16

FOR

DRESDEN NUCLEAR POWER STATION

COMMONWEALTH EDISON COMPANY

FOR JANUARY 1997

<u>UNIT</u>	<u>DOCKET</u>	<u>LICENSE</u>
1	050-010	DPR-2
2	050-237	DPR-19
3	050-249	DPR-25

TABLE OF CONTENTS
JANUARY 1997 NRC REPORT

- 1.0 Introduction
- 2.0 Summary of Operating Experience
 - 2.1 Unit 2 Monthly Operating Experience Summary.
 - 2.2 Unit 3 Monthly Operating Experience Summary.
- 3.0 Operating Data Statistics
 - 3.1 Operating Data Report - Dresden Unit 2
 - 3.2 Operating Data Report - Dresden Unit 3
 - 3.3 Average Daily Unit 2 Power Level
 - 3.4 Average Daily Unit 3 Power Level
 - 3.5 Unit 2 Shutdowns and Power Reductions
 - 3.6 Unit 3 Shutdowns and Power Reductions
- 4.0 Unique Reporting Requirements
 - 4.1 Main Steam Relief and/or Safety Valve Operations - Unit 2 and Unit 3
 - 4.2 Off-Site Dose Calculation Manual Changes
 - 4.3 Major Changes to the Radioactive Waste Treatment Systems
 - 4.4 Failed Fuel Element Indications
 - 4.4.1 Unit 2
 - 4.4.2 Unit 3
- 5.0 Technical Specification Amendments
 - 5.1 Amendments to Facility License or Technical Specifications
 - 5.1.1 Unit 2
 - 5.1.2 Unit 3

1.0 Introduction

Dresden Nuclear Power Station is a three reactor generating facility owned and operated by the ComEd Company of Chicago, Illinois. Dresden Station is located at the confluence of the Kankakee and Des Plaines Rivers, in Grundy County, near Morris, Illinois.

Dresden Unit 1 is a General Electric Boiling Water Reactor with a design net electrical output rating of 200 megawatts electrical (MWe). The unit is retired in place with all nuclear fuel removed from the reactor vessel. Therefore, no Unit 1 operating data is provided in this report.

Dresden Units 2 and 3 are General Electric Boiling Water Reactors, each licensed at 2527 megawatts thermal. The gross outputs of Units 2 and 3 are 832 and 834 megawatts electrical, respectively, with design net electrical output ratings of 794 MWe each. The commercial service date for Unit 2 is 11 August 1970, and 30 October 1971 for Unit 3.

Waste heat is rejected to a man-made cooling lake using the Kankakee River for make-up and the Illinois River for blowdown.

The Architect-Engineer for Dresden Units 2 and 3 was Sargent and Lundy of Chicago, Illinois.

This report for **January 1997**, was compiled by Gary A. Abrell of Dresden Regulatory Assurance Staff, telephone number (815) 942-2920, extension 3749.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR January 1997

2.1 UNIT 2 MONTHLY OPERATING EXPERIENCE SUMMARY

Unit 2 was on system at the beginning of the period at approximately 300 MWE while trouble shooting #1 Turbine Control Valve. At 1815 on January 3, 1997, the control valve was successfully reopened. At 1859 on January 4, 1997, commenced load pick-up and achieved full power at 1330 on January 6, 1997.

The unit remained at full power the remainder of the period except for minor load drops for surveillance and testing.

3.0 SUMMARY OF OPERATING EXPERIENCE FOR January 1997

2.2 UNIT 3 MONTHLY OPERATING EXPERIENCE SUMMARY

Unit 3 was shutdown at the beginning of the period for repairs to the 3B recirculating pump motor and completion of other outage tasks.

On December 20, 1996, an error was discovered in emergency core cooling analysis calculation. The revised calculation revealed a deficit in the net positive suction head required for the ECCS pumps during the design basis accident. An emergency technical specification amendment was submitted on January 13, 1997 to permit restart of the unit. The amendment was received on January 28, 1997.

At 2314 on January 29, 1997, the reactor was made critical and was synchronized to the grid at 1444 on January 31, 1997.

3.0 OPERATING DATA STATISTICS

3.1 OPERATING DATA REPORT - DRESDEN UNIT TWO

DOCKET No. 050-237
DATE February 4, 1997
COMPLETED BY G. A. ABRELL
TELEPHONE (815) 942-2920

OPERATING STATUS

1. REPORTING PERIOD: **January 1997**
2. CURRENTLY AUTHORIZED POWER LEVEL (MWth): 2,527
MAXIMUM DEPENDABLE CAPACITY (MWe NET): 772
DESIGN ELECTRICAL RATING (MWe Net): 794
3. POWER LEVEL TO WHICH RESTRICTED (MWe Net): No restrictions
4. REASONS FOR RESTRICTIONS (IF ANY): See Section 2.1 of this report.

REPORTING PERIOD DATA			
PARAMETER	THIS MONTH	YEAR TO DATE	CUMULATIVE
HOURS IN PERIOD	744	744	232,080
TIME REACTOR CRITICAL	744	744	167,906
TIME REACTOR RESERVE SHUTDOWN (Hours)	0	0	0
TIME GENERATOR ON-LINE (Hours)	744	744	159,759
TIME GENERATOR RESERVE SHUTDOWN (Hours)	0	0	1
THERMAL ENERGY GENERATED (MWh Gross)	1,686,231	1,686,231	333,390,132
ELECTRICAL ENERGY GENERATED (MWEHe Gross)	549,061	549,061	106,446,997
ELECTRICAL ENERGY GENERATED (MWEHe Net)	526,180	526,180	100,627,909
REACTOR SERVICE FACTOR (%)	100.0%	100.0%	72.3%
REACTOR AVAILABILITY FACTOR (%)	100.0%	100.0%	72.3%
GENERATOR SERVICE FACTOR (%)	100.0%	100.0%	68.8%
GENERATOR AVAILABILITY FACTOR (%)	100.0%	100.0%	68.8%
CAPACITY FACTOR (Using MCD Net) (%)	91.6%	91.6%	56.2%
CAPACITY FACTOR (Using DER Net) (%)	89.1%	89.1%	54.6%
FORCED OUTAGE FACTOR (%)	0%	0.0%	12.6%

20. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS: NONE
21. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

3.0 OPERATING DATA STATISTICS

3.2 OPERATING DATA REPORT - DRESDEN UNIT THREE

DOCKET No. 050-249
 DATE February 4, 1997
 COMPLETED BY G. A. ABRELL
 TELEPHONE (815) 942-2920

OPERATING STATUS

1. REPORTING PERIOD: January 1997
2. CURRENTLY AUTHORIZED POWER LEVEL (MWth): 2,527
 MAXIMUM DEPENDABLE CAPACITY (MWe Net): 773
 DESIGN ELECTRICAL RATING (MWe Net): 794
3. POWER LEVEL TO WHICH RESTRICTED: No restriction
4. REASONS FOR RESTRICTIONS (IF ANY): See Section 2.2 of this report.

REPORTING PERIOD DATA			
PARAMETER	THIS MONTH	YEAR TO DATE	CUMULATIVE
HOURS IN PERIOD	744	744	221,400
TIME REACTOR CRITICAL	48.8	48.8	155,906
TIME REACTOR RESERVE SHUTDOWN (Hours)	0.0	0	0
TIME GENERATOR ON-LINE (Hours)	10.0	10	148,534
TIME GENERATOR RESERVE SHUTDOWN (Hours)	0.0	0	0
THERMAL ENERGY GENERATED (MWh Gross)	1,636	1,636	310,214,113
ELECTRICAL ENERGY GENERATED (MWe Gross)	1,320	1,320	99,274,461
ELECTRICAL ENERGY GENERATED (MWe Net)	-4,648	-4,648	94,077,853
REACTOR SERVICE FACTOR (%)	6.6%	6.6%	70.4%
REACTOR AVAILABILITY FACTOR (%)	6.6%	6.6%	70.4%
GENERATOR SERVICE FACTOR (%)	1.3%	1.3%	67.1%
GENERATOR AVAILABILITY FACTOR (%)	1.3%	1.3%	67.1%
CAPACITY FACTOR (Using MCD Net) (%)	-0.8%	-0.8%	55.0%
CAPACITY FACTOR (Using DER Net) (%)	-0.8%	-0.8%	53.5%
FORCED OUTAGE FACTOR (%)	98.7%	98.7%	14.8%

20. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS: March 29, 1997 for D3R14.
21. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

3.3 AVERAGE DAILY UNIT 2 POWER LEVEL

DOCKET No. 050-237

UNIT Dresden 2

DATE February 4, 1997

COMPLETED BY G. A. ABRELL

TELEPHONE (815) 942-2920

MONTH: January-97

DRESDEN 2

DAY	AVERAGE DAILY NET POWER LEVEL (MWe)	DAY	AVERAGE DAILY NET POWER LEVEL (MWe)
1	285	17	792
2	286	18	792
3	278	19	790
4	262	20	793
5	299	21	793
6	743	22	792
7	787	23	791
8	792	24	791
9	794	25	792
10	793	26	790
11	792	27	792
12	791	28	791
13	792	29	791
14	792	30	787
15	791	31	787
16	784		

(Note: negative values represent station loads)

3.4

AVERAGE DAILY UNIT 3 POWER LEVEL

DOCKET No. 050-249

UNIT Dresden 3

DATE February 7, 1997

COMPLETED BY G. A. ABRELL

TELEPHONE (815) 942-2920

MONTH: January-97

DRESDEN 3

DAY	AVERAGE DAILY NET POWER LEVEL (MWe)	DAY	AVERAGE DAILY NET POWER LEVEL (MWe)
1	-8	17	-8
2	-8	18	-8
3	-8	19	-8
4	-8	20	-8
5	-8	21	-8
6	-8	22	-8
7	-8	23	-8
8	-8	24	-8
9	-8	25	-8
10	-8	26	-8
11	-8	27	-8
12	-8	28	-8
13	-8	29	-8
14	-8	30	-8
15	-8	31	46
16	-8		

(Note: Negative values represent station loads)

3.5 UNIT 2 SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OF January 1997

NO	DATE	TYPE (1)	DURATION (HOURS)*	REASON (2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE (4)	COMPO- NENT CODE (5)	CORREC- TIVE ACTIONS/ COM- MENTS
10	961220	S	0	B/A	5	N/A	N/A	N/A	2.1

Year-to-date forced outage hours = 0

Cumulative forced outage hours = 22,935

TABLE KEY:

(1)

F: Forced
S: Scheduled

(2)

Reason:

A Equipment Failure (Explain)
B Maintenance or Test
C Refueling
D Regulatory Restriction
E Operator Training &
Licensing Exam
F Administrative
G Operational Error
H Other (Explain)

(3)

Method:

1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)
5. Load Reduction

(4)

Exhibit G Instruction for
Preparation of Data Entry
Sheets for Licensee Event
Reports (LER) File
(NUREG-0161)

(5)

Exhibit I Same Source as
Above.

3.6 UNIT 3 SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January 1997

NO	DATE	TYPE (1)	DURATION (HOURS) *	REASON (2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE (4)	COMPONENT CODE (5)	CORREC- TIVE ACTIONS/ COMMENTS
4	961027	F	734	A	1	N/A	AD	MO	2.2

Year-to-date forced outage hours = 734

Cumulative forced outage hours = 25,810

TABLE KEY:

(1)

F: Forced
S: Scheduled

(2)

Reason:

A Equipment Failure
(Explain)
B Maintenance or Test
C Refueling
D Regulatory Restriction
E Operator Training &
Licensing Exam
F Administrative
G Operational Error
H Other (Explain)

(3)

Method:

1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)
5. Load Reduction

(4)

Exhibit G Instruction for
Preparation of Data Entry
Sheets for Licensee Event
Reports (LER) File
(NUREG-0161)

(5)

Exhibit I Same Source as
Above.

4.0 UNIQUE REPORTING REQUIREMENTS

4.1 MAIN STEAM RELIEF AND/OR SAFETY VALVE OPERATIONS - UNIT 2 AND UNIT 3

Unit 3 3D electromatic relief valve was tested at low pressure on January 31, 1997.

4.2 OFF-SITE DOSE CALCULATION MANUAL (ODCM) CHANGES

In addition to numerous editorial changes, the following revisions were made.

- 12-13 Changed requirements for the Reactor Building Vent Exhaust Duct Radiation Monitor minimum channels to say "See Technical Specifications Section 3/4.2". This will ensure the user is cognizant that this item appears in the Technical Specifications.
- 12-14 Deleted 7 day clock and reporting requirement for Action 20. Upgraded Tech Specs do not address Low Range 2(3) Chimney noble gas monitoring and change is consistent with current Tech Specs and Quad Cities and LaSalle ODCM requirements for comparable monitors. Deleted Action 24 associated with the Reactor Building Vent Exhaust Duct Radiation Monitor (see change for page 12-13).
- 12-16 Offgas monitor Functional Test frequency changed from daily to quarterly and Instrument check changed from quarterly to daily to comply with current Tech Specs.
- 11-2 Removed D-05 and footnote. D-05 deleted in 1995 as indicated by the footnote. Removed asterisk by D-45 indicating it replaced D-05 in 1995.
- 11-6 Added D-46 as a unit 2/3 Cooling Water Sample location.
- 11-14 Removed an incorrect milk location from the map; two locations were noted for D-26.

4.3 MAJOR CHANGES TO THE RADIOACTIVE WASTE TREATMENT SYSTEMS

None

4.4 FAILED FUEL ELEMENT INDICATIONS

4.4.1 Unit 2

Unit 2 has no indications of fuel failures.

4.4.2 Unit 3

Unit 3 Previous operation indicated a single fuel rod failure.

5.0 TECHNICAL SPECIFICATION AMENDMENTS

5.1 Amendments to Facility License or Technical Specifications implemented during January 1997.

January 13, 1997 implemented the Upgraded Technical Specifications (TSUP) for Units 2 and 3 except for the section dealing with control room ventilation which was implemented on January 15, 1997.

January 28, 1997 received amendment 152 to DPR 19 and amendment 147 to DPR 25. The amendment evaluated an unreviewed safety question as a result of an error discovered in the calculation of head loss across the Emergency Core Cooling Suction Strainers. The amendment lowered the allowable torus water temperature and the ultimate heat sink temperature. In addition it changed the basis of the technical specification to allow credit for two pounds containment overpressure during the first ten minutes of the Design Basis Accident. These changes compensated for the deficiency in the net positive suction head available to the ECCS.

The new limits on torus water and ultimate heat sink temperature will not be achievable during the late spring and summer months. Therefore, a new amendment will be necessary to restart the unit following the refueling outage.