

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Browns Ferry - Unit 3DOCKET NUMBER (2)
0 5 0 0 0 2 9 6 1 OF 0 1 2

PAGE (3)

TITLE (4)
Surveillance Testing Revealed Less Than Minimum Radiation Monitors Operable

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIA NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)	
0	7	2	5	8	5	0	1	9	Browns Ferry - Unit 1	0 5 0 0 0 2 5 9	
0	7	2	5	8	5	0	0	8	Browns Ferry - Unit 2	0 5 0 0 0 2 6 0	

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)									
POWER LEVEL (10)	0 1 0 1 0	20.402(b)		20.405(c)		<input checked="" type="checkbox"/> 50.73(a)(2)(iv)		73.71(b)			
		20.405(a)(1)(i)		50.38(c)(1)		50.73(a)(2)(v)		73.71(c)			
		20.405(a)(1)(ii)		50.38(c)(2)		50.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 365A)			
		20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(vii)(A)					
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(vii)(B)					
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
David L. Smith	2 0 5 7 2 9 - 3 8 6 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input checked="" type="checkbox"/>	<input type="checkbox"/>				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During performance of surveillance testing on the reactor building ventilation exhaust radiation monitors, an inadvertent secondary containment isolation occurred, and later both reactor zone radiation monitors trip setpoints were found to have drifted above technical specification limits. The inadvertent isolation cause was determined, the systems were returned to normal, and the surveillance testing was continued. Later during performance of the same surveillance testing, the two reactor zone exhaust radiation monitor setpoints were reset to technical specification limits, and the surveillance test satisfactorily completed.

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PDR ADOCK 05000296
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Browns Ferry - Unit 3	0500029685	85	019	00	02	OF	02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Units 1 and 2 were in refueling outages, and unit 3 was in an extended maintenance outage.

On July 25, 1985, (approximately 0855) while performing Surveillance Instruction (SI) 4.2.A.10, Reactor Building Ventilation Radiation Monitors (IL), a voltmeter was being hooked to the 24 Vdc radiation monitors power supply, and an arc occurred causing both monitors (MTR) to momentarily go downscale. This downscale initiated the expected engineering safeguards functions. The surveillance procedure contains a precaution that either two downscales or one upscale in one channel would actuate five different engineering safeguards functions. The required safety system actuations were received as designed (refuel zone isolation, unit 3 reactor zone isolation, partial primary containment isolation system (JM) isolation, standby gas treatment (VA) initiation, and control room emergency ventilation (VI) initiation). After verification that no abnormal situation existed, the licensed reactor operator returned the affected safety systems to standby readiness.

The arc was caused by improper voltmeter hookup techniques. A precaution is being added to the applicable surveillance test instruction step to preclude this type occurrence.

Also during the performance of SI 4.2.A.10 on July 25, 1985, (approximately 1445), plant personnel found two reactor zone ventilation exhaust radiation monitors that had drifted above the technical specification limit. The required setpoint is less than or equal to 100 mr/hr. The monitors were found to trip at 110 mr/hr (RM-90-142) and 150 mr/hr (RM-90-143). The calibration history for both monitors was reviewed, and setpoints drifting in the nonconservative direction has not been experienced within the past two years.

Although the radiation monitors were above the technical specification limit of 100 mr/hr, the same release path is also monitored by continuous air monitors (MON) which have alarming functions in the control rooms. Upon receiving a continuous air monitor alarm, the licensed reactor operator is directed by operating procedures to verify the alarm and then isolate the applicable release path(s) if the alarm is valid.

The setpoints were immediately returned to the technical specification limit of less than or equal to 100 mr/hr, and the surveillance test was successfully completed. An evaluation of the electrical arc was performed and judged not to have caused the out of limit setpoints. The SI was performed again on August 8, 1985, and the setpoint drift in a nonconservative direction was not observed. No further corrective action is planned.

Responsible Plant Section - N/A

Previous Events - None

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant

P. O. Box 2000

Decatur, Alabama 35602

August 23, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

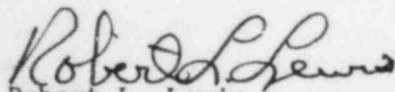
Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 3 -
DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE
OCCURRENCE REPORT BFRO-50-296/85019

The enclosed report provides details concerning ventilation exhaust
radiation monitor surveillance testing which revealed less than minimum
monitors operable. This report is submitted in accordance with
10 CFR 50.73(a)(2)(iv).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



Robert L. Lewis
Acting Plant Manager
Browns Ferry Nuclear Plant

Enclosures

cc (Enclosures):

Regional Administrator
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
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INPO Records Center
Suite 1500
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NRC Resident Inspector, BFN

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