



GPU Nuclear Corporation

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August 27, 1985

Mr. John A. Zwolinski, Chief
Operating Reactors Branch No. 5
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Mr. Zwolinski:

Subject: Oyster Creek Nuclear Generating Station (OCNGS)
Docket No. 50-219
SEP Topic No. VI-4, Containment Isolation System

The Containment Spray and Core Spray systems are closed systems and are provided with remote manual isolation valves. Since operator action is required to initiate isolation, if necessary, the operator must know when to initiate isolation. This requires a leakage detection capability and appropriate procedures to indicate under what conditions these valves should be shut. During the integrated assessment of the subject SEP topic, the NRC staff requested GPUN to evaluate the leakage detection provisions, provide appropriate procedures for operator action and relocate the operating station to an accessible area, when necessary, for the following valves:

V-20-3	V-21-5
V-20-4	V-21-7
V-20-32	V-21-9
V-20-33	V-21-11
V-21-1	V-21-15
V-21-3	V-21-18

GPUN has reviewed each of the valves listed above and determined that all of the valves listed can be operated from the 460 V switchgear room located in the Office Building which is assessible during post accident conditions.

Should a failure occur in any of the Containment or Core Spray lines outside the containment there are indications to alert the operator. These indications include pressure, flow indications and sump pump running indications. The attached table identifies the specific indications that would be expected for a break in either the Core Spray or Containment Spray systems. Although an excess flow with low discharge pressure condition would be observed following a break downstream of the flow element in either system,

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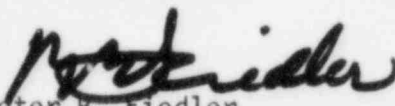
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the same indications would be seen when these systems are being utilized to mitigate an accident. Therefore, it is not possible to distinguish a cause based on these indications. However, both the Core and Containment Spray lines are low energy piping (low pressure and low temperature) and it is highly unlikely to experience a line break between the flow element and the containment wall in conjunction with an accident which requires Core Spray and/or Containment Spray actuation.

Based on the above, sufficient indications presently exist to alert the operator of the need to isolate the affected system in the event of a break between the suppression chamber and the flow elements in both systems (major portion of the piping). As it was stated earlier, all of the valves can be operated from the 460 V switchgear room which is accessible at all times.

Plant procedures will be revised to incorporate isolation provisions with operator action before restart from the Cycle 11 refueling outage.

Very truly yours,


Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF/YN/dam
Attachment
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cc: Dr. Thomas E. Murley, Administrator
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NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731

SYSTEM

DISCHARGE LINE BREAK

SUCTION LINE BREAK

Containment Spray	<p><u>Indication:</u></p> <p>No flow or very reduced flow, low pressure (Break before flow elements FE 1P02A, B)</p>	<p><u>Indications:</u></p> <p>Reduced differential pressure between tube side and shell of heat exchangers</p> <p>Temperature elements, TE 40B, C will show decrease in temperature because of increased cooling due to reduced flow.</p> <p>Sump pump running indication</p>
Core Spray	<p><u>Indication:</u></p> <p>Low discharge pressure, no flow (break before flow elements FE-RV25A, B)</p>	<p><u>Indications:</u></p> <p>Loss of pressure (break occurs before suction pressure indicators PI-25A, B, C, D - local indicator)</p>