NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Fermi-2	0 5 0 0 0 3 4 1	1 OF 0 1

TITLE (4)
Reactor Water Clean-up Delta-Flow Instrumentation Surveillance Inadequacies

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
												0 5 0 0 0
0 7	2 0	8 5	8 5	-	0 4 0	-	0 0	0 8	2 4	8 5		0 5 0 0 0

OPERATING MODE (9)	2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)					
POWER LEVEL (10)	0 1 0 2	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)		
		20.405(a)(1)(i)	50.36(c)(1)	X 50.73(a)(2)(v)	73.71(c)		
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)			
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)			
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)			

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
L.P. Bregni, Compliance Engineer	AREA CODE 3 1 3 5 8 6 - 5 3 1 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 21, 1985 while performing a channel functional test on the Reactor Water Cleanup (RWCU) system differential flow instrumentation, an error was discovered in the surveillance procedure. The instrument repairman performing the test recognized that the test procedure did not adequately determine the isolation trip setpoint. The plant was in Operatiohal Condition 2 and reactor power was about two percent at the time.

Upon investigation it was determined that an earlier procedure revision had omitted a step requiring the measurement and recording of the test input signal which is varied to cause a trip actuation. This information is required to correctly determine the actual process parameter value at which the instrumentation causes the isolation valve to close.

The procedure was modified to measure and record the input signal and the instrument loop was recalibrated. The calibaration indicated the setpoint to be within the technical specification allowable value. Steam leak detection circuitry which is also designed to sense an RWCU system leak or break was functional at the time and capable of isolating such a break if necessary, hence no safety significance is attached to this event.

IE22
1/1

NRC Form 366
(9-83)

Detroit Edison
2000 Second Avenue
Detroit, Michigan 48226
(313) 237-8000

August 24, 1985
NP850041

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Reference: Fermi 2
NRC Docket No. 50-341
NRC Operating License No. NPF-43

Subject: Transmittal of Licensee Event Report 85-040

Please find enclosed LER No. 85-040-00, dated August 24, 1985, for a reportable event which occurred on July 20, 1985. Region III was notified on August 19, 1985 that LER 85-040 would be submitted on August 26, 1985. As indicated below, a copy of this LER is being sent to the Administrator Region III.

If you have any questions, please contact us.

Sincerely,

R. S. Lenart
Plant Manager

Enclosure: NRC Forms 366

cc: P.M. Byron
M.D. Lynch

Regional Administrator
USNRC Region III
799 Roosevelt Rd.
Glen Ellyn, IL 60137

Director/Coordinator
Monroe City-County Office of Civil Preparedness
965 South Raisinville Road
Monroe, MI 48161

IE22
1/1