

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Dresden Nuclear Power Station, Unit 3										DOCKET NUMBER (2) 0 8 0 0 0 2 4 9 1										PAGE (3) 0 2							
TITLE (4) Reactor Scram																											
EVENT DATE (6)			LER NUMBER (8)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (5)																
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME							DOCKET NUMBER (5)											
0	3	2	6	8	4	8	4	—	0	0	2	—	0	0	0	4	2	4	8	4	0 8 0 0 0						
OPERATING MODE (9)			THE REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 43. (Check one or more of the following) (11)																								
N			<div style="display: flex; justify-content: space-between;"> <div> 90.400(b) 90.400(a)(1)(H) 90.400(a)(1)(B) 90.400(a)(1)(C) 90.400(a)(1)(D) 90.400(a)(1)(E) </div> <div> 90.400(c) 90.300(d)(1) 90.300(d)(2) 90.300(d)(3) 90.300(d)(4) 90.300(d)(5) </div> <div> <input checked="" type="checkbox"/> 90.700(a)(2)(H) <input type="checkbox"/> 90.700(a)(2)(I) <input type="checkbox"/> 90.700(a)(2)(J) <input type="checkbox"/> 90.700(a)(2)(K) <input type="checkbox"/> 90.700(a)(2)(L) <input type="checkbox"/> 90.700(a)(2)(M) <input type="checkbox"/> 90.700(a)(2)(N) </div> <div> 90.700(b) 90.700(c) OTHER (Specify in Abstract below and in Part 3 of Form 300A) </div> </div>																								
POWER LEVEL (10) 0 0 0																											
LICENSEE CONTACT FOR THIS LER (13)																											
NAME Gerald W. Bergan (X529)												TELEPHONE NUMBER AREA CODE 8 1 5 9 4 2 - 2 9 2 0															
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (12)																											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC																		
X																											
SUPPLEMENTAL REPORT EXPECTED (14)																											
YES (If yes, complete EXPECTED SUBMISSION DATE)										NO										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR			
										X																	

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMS NO 3180-0104

EXPIRES 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (3)				PAGE (3)															
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER																	
		0	8	0	0	0	2	4	9	8	4	-	0	0	2	-	0	0	0	2	OF

TEXT (If more space is required, use additional NRC Form 285a-1/ (17))

With the reactor in refuel, an Instrument Mechanic was sent to perform maintenance on the preamplifier of Intermediate Range Monitor (IRM) #18. The IRM's are part of the reactor power monitoring system whose preamplifiers are housed in a metal cabinet in the reactor building. Included in the cabinet with the preamplifier of IRM #18 are the preamplifiers of IRM #'s 12, 14 and 16. While doing maintenance, IRM #14 (Channel A) and IRM #16 (Channel B) spiked hi-hi (120/125 full scale) causing a full reactor scram. All required protective systems performed as designed and the scram signals were immediately reset. This event was repeated 16 minutes later.

The cause of this event is attributed to electronic noise entering the cabinet while the door was opened and inducing spurious signals in the IRM circuitry. The source of the noise is undetermined. A previous event was reported by DVR 12-2-83-98.



Commonwealth Edison

DEVIATION REPORT

DOR NO. 12 - 3 - 84 - 11
STA UNIT YEAR NO.

PART 1 TITLE OF DEVIATION

Reactor Scram

OCCURRED 3-26-84 1409 and
DATE TIME 1425SYSTEM AFFECTED 0500
RPS

PLANT STATUS AT TIME OF EVENT

MODE Refuel POWER(%) 0 %

TESTING

☐ YES ☒ NO

DESCRIPTION OF EVENT

With I.M. Department performing maintenance on IRM 18 preamp, compartment door housing IRM preamps for IRM's 12, 14, 16, and 18 was necessarily opened. This allowed a spurious electronic noise to enter the IRM circuitry and IRM 14 (Channel A) and IRM 16 (Channel B) tripped on flux Hi Hi causing a scram. Same cause resulted in a second scram 16 minutes later.

POTENTIALLY SIGNIFICANT EVENT PER NSD DIRECTIVE A-07

☐ YES ☒ NO10CFR50.73 MRC RED PHONE
NOTIFICATION MADE☒ 1 HOUR
☐ 4 HOUR

1522

☐ NO

TIME

RESPONSIBLE SUPERVISOR Korchynsky

DATE 3-26-84

PART 2 OPERATING ENGINEER'S COMMENTS

All control rods were fully inserted at the time of the scram. All systems performed as designed and the scram signals were immediately reset.

☐ NON REPORTABLE EVENT☒ 30 DAY REPORTABLE/
10CFR 50.73 (A)(2)(iv)☐ 5 DAY REPORT PER 10CFR21☐ ANNUAL/SPECIAL REPORT REQUIRED
PER

A.I.R. #

L.E.R. # 84-002-0

NOTIFICATION N/A
REGION III

DATE TIME

TELECOPY Dennis P. Galle
NSD4-6-84 1518
DATE TIME☐ CECO CORPORATE NOTIFICATION MADE
IF ABOVE NOTIFICATION IS PER 10CFR21

TELEPHONE N/A

CECO CORPORATE OFFICER

DATE TIME

PRELIMINARY REPORT
COMPLETED AND REVIEWED

T. A. Ciesla

4-5-84

OPERATING ENGINEER

DATE

INVESTIGATION REPORT & RESOLUTION
ACCEPTED BY STATION REVIEWRESOLUTION APPROVED AND
AUTHORIZED FOR DISTRIBUTION

(Form 15-52-1)

1-84

STATION SUPERINTENDENT

DATE



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/842-2920

RAN
5-10-84

April 24, 1984

000 1137 B05

DJS Ltr. #84-410

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

WSG
1/G G

LER	
LER #	249-84002
EVENT DATE	840321
INPO RCVD DATE	840508
NSAC RCVD DATE	

Licensee Event Report #84-002-0, Docket #050-249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73(A)(2)(iv).

E. M. Scott

D. J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/jmt

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

IE 22
11
'84 RETROFIT