February 13, 1997

Mr. Charles H. Cruse
Vice President - Nuclear Energy
Balitmore Gas and Electric Company
Calvert Cliffs Nuclear Power Plant
1650 Calvert Cliffs Parkway
Lusby, MD 20657-4702

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - PROPOSED TECHNICAL SPECIFICATION CHANGES TO REACTOR COOLANT SYSTEM FLOW LIMIT, CALVERT CLIFFS NUCLEAR POWER PLANT, UNITS 1 AND 2 (TAC NOS. M97855 AND M97856)

Dear Mr. Cruse:

By letter dated January 31, 1997, the Baltimore Gas and Electric Company, the licensee, pursuant to 10 CFR 50.90, requested an amendment to Facility Operating License DPR-53 and DPR-69 for the Calvert Cliffs, Units 1 and 2, respectively. The amendment proposed a modification to Technical Specifications for a reduction of the total reactor coolant system flow rate lower limit by 30,000 gallons per minute (gpm) from 370,000 gpm. The NRC staff is reviewing your submittal and has determined that additional information is required for us to complete our review. The information requested is addressed in the enclosure. In order to meet your schedule, the staff requests that the additional information be provided by March 1, 1997.

If you have any questions regarding this matter, please contact me at 301-415-3473.

Sincerely,
/S/
Alexander W. Dromerick, Senior Project Manager
Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-317
and 50-318

Enclosure: Request for Additional Information
cc w/encl: See next page
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 13, 1997

Mr. Charles H. Cruse
Vice President - Nuclear Energy
Baltimore Gas and Electric Company
Calvert Cliffs Nuclear Power Plant
1650 Calvert Cliffs Parkway
Lusby, MD 20657-4702

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - PROPOSED TECHNICAL SPECIFICATION CHANGES TO REACTOR COOLANT SYSTEM FLOW LIMIT, CALVERT CLIFFS NUCLEAR POWER PLANT, UNITS 1 AND 2 (TAC NOS. M97855 AND M97856)

Dear Mr. Cruse:

By letter dated January 31, 1997, the Baltimore Gas and Electric Company, the licensee, pursuant to 10 CFR 50.90, requested an amendment to Facility Operating License DPR-53 and DPR-69 for the Calvert Cliffs, Units 1 and 2, respectively. The amendment proposed a modification to Technical Specifications for a reduction of the total reactor coolant system flow rate lower limit by 30,000 gallons per minute (gpm) from 370,000 gpm. The NRC staff is reviewing your submittal and has determined that additional information is required for us to complete our review. The information requested is addressed in the enclosure. In order to meet your schedule, the staff requests that the additional information be provided by March 1, 1997.

If you have any questions regarding this matter, please contact me at 301-415-3473.

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Project Directorate I-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-317
and 50-318

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cc w/encl: See next page

REQUEST FOR ADDITIONAL INFORMATION
PROPOSED AMENDMENT FOR REDUCED RCS FLOW
CALVERT CLIFFS NUCLEAR POWER PLANT, UNITS 1 AND 2

Reference: Baltimore Gas and Electric Company letter to NRC, dated January 31, 1997, "License Amendment Request; Change to Reactor Coolant System Flow Requirements to Allow Increased Steam Generator Tube Plugging."

1. On page 1 of Attachment 1 of the transmittal, it is stated that "The primary effects of plugging Steam Generator (SG) tubes are to reduce SG heat transfer area and increase the Reactor Coolant System (RCS) flow resistance." These effects result in reduced SG steam pressure, reduced RCS flow rate, and increase hot leg temperature. Provide a comparison of key parameters (e.g. RCS pressure, T average, T cold, T hot, steam pressure, SG outlet temperature) between the proposed operation and the design basis analysis.

2. The decrease in steam pressure results in the increase in the primary-to-secondary pressure difference and break flow. Provide evaluation of the effect of decreased steam pressure on the design basis analysis of SGs with respect to the stresses and fatigue usage.

3. On page 2 of the transmittal, you stated that the reduced RCS flow rate results in an increase in the temperature rise (delta-T) between the cold leg and the hot leg. Provide evaluation of the effect of increased delta-T on the reactor vessel and internals, piping, the CRDM housing, the pressurizer (lower head and upper shell), surge line (stratification), pressurizer spray nozzles, and the reactor coolant pumps, with respect to stresses and fatigue analysis.

Enclosure

Mr. Charles H. Cruse
Calvert Cliffs Nuclear Power Plant

Calvert Cliffs Nuclear Power Plant
Unit Nos. 1 and 2

cc:

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