



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

OMAHA PUBLIC POWER DISTRICT

DOCKET NO. 50-285

FORT CALHOUN STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 90  
License No. DPR-40

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Omaha Public Power District (the licensee) dated June 11, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, Facility Operating License No. DPR-40 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-40 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 90, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 30 days of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Edward J. Butcher, Acting Chief  
Operating Reactors Branch #3  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 19, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 90

FACILITY OPERATING LICENSE NO. DPR-40

DOCKET NO. 50-285

Revise the Appendix "A" Technical Specifications as indicated below. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages

3-62  
3-62a

Insert Pages

3-62  
3-62a

### 3.0 SURVEILLANCE REQUIREMENTS

#### 3.9 Auxiliary Feedwater System

##### Applicability

Applies to periodic testing requirements of the turbine-driven and motor-driven auxiliary feedwater pumps.

##### Objective

To verify the operability of the auxiliary feedwater (AFW) system and its ability to respond properly when required.

##### Specifications

- (1) The position of valves necessary to ensure auxiliary feedwater flow to the steam generators shall be verified by a monthly inspection. Anytime maintenance is performed on the auxiliary feedwater system which alters valve alignments, an operator shall check that the AFW system valves are properly aligned, to ensure AFW flow to the steam generators, and a second operator shall independently verify proper valve alignment.
- (2) The operability of the motor-driven auxiliary feedwater pump and the steam turbine-driven auxiliary feedwater pump shall be confirmed at least monthly.
- (3) The operability of auxiliary feedwater pumps' steam generator level regulating valves HCV-1107A, HCV-1107B, HCV-1108A, HCV-1108B, and auxiliary feedwater cross-tie valve HCV-1384 shall be confirmed at least every three months.
- (4) The capabilities of the motor-driven and turbine-driven auxiliary feedwater pumps shall be verified by using local pressure indicators and flow indicators in the control room. The discharge pressure will be verified to be 40 psig above the steam generator pressure at rated steam flow.
- (5) Following cold shutdown and prior to raising the reactor coolant temperature above 300°F, the motor-driven auxiliary feedwater pump shall be tested to verify the normal flow path for auxiliary feedwater to the steam generators.
- (6) At least once per 18 months during shutdown by:
  - a. Verifying that each automatic valve in the flow path actuates to its correct position upon receipt of each auxiliary feedwater actuation test signal.
  - b. Verifying that each auxiliary feedwater pump starts as designed automatically upon receipt of each auxiliary feedwater actuation test signal.

### 3.0 SURVEILLANCE REQUIREMENTS

#### 3.9 Auxiliary Feedwater System (Continued)

##### Basis

The valve position verifications performed monthly and following auxiliary feedwater system maintenance will confirm the availability of an auxiliary feedwater flow path to the steam generators.

The testing of the auxiliary feedwater pumps every month and after cold shutdowns will verify their operability by recirculating water to the emergency feedwater storage tank.

Operating the regulating valves (HCV-1107A, HCV-1107B, HCV-1108A and HCV-1108B) one at a time every three months and after cold shutdowns will confirm a flow path to the steam generators and operability of the valves.

Proper functioning of the steam turbine admission valve and starting of the feedwater pump will demonstrate the integrity of the steam driven pump. Verification of correct operation will be made both from instrumentation within the main control room and direct visual observation of the pumps.

The operability of the auxiliary feedwater system ensures that the reactor coolant system can be cooled down to less than 350°F from normal operating conditions in the event of a total loss of off-site power.

##### References

- (1) FSAR, Section 9.4
- (2) Technical Specification 2.5