



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 18, 1996

Mr. Joesph J. Hagan
Vice President, Operations GGNS
Entergy Operations, Inc.
P. O. Box 756
Port Gibson, MS 39150

SUBJECT: RELIEF FROM THE SECTION XI TESTING REQUIREMENTS IN THE ASME CODE FOR
SAFETY/RELIEF VALVES AT GRAND GULF NUCLEAR STATION, UNIT 1
(TAC NO. M95403)

Dear Mr. Hagan:

The staff has reviewed the information provided in your letter of May 9, 1996, for Grand Gulf Nuclear Station, Unit 1 (GGNS), related to (1) a proposed change to the surveillance requirements in the plant Technical Specifications (TSs) and (2) an associated request for relief from the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code) inservice testing (IST) requirements for the main steam safety and relief valves (S/RVs). This letter concerns the request for relief from Section XI of the ASME Code for these valves. The proposed change to the TSs will be addressed in a separate letter.

The Code of Federal Regulations, 10 CFR 50.55a, requires that IST of certain ASME Code Class 1, 2 and 3 pumps and valves be performed in accordance with Section XI of the ASME Code and applicable addenda, except where relief has been requested and granted, or proposed alternatives have been authorized, by the Commission pursuant to Paragraphs (a)(3)(i), (a)(3)(ii), or (f)(6)(i) of 10 CFR 50.55a. In proposing alternatives or requesting relief, the applicant must demonstrate that: (1) the proposed alternative provides an acceptable level of quality and safety (Paragraph (a)(3)(i)); (2) compliance would result in a hardship or unusual difficulty without a compensating increase in the level of quality and safety (Paragraph (a)(3)(ii)); or (3) conformance is impractical for the facility (Paragraph (f)(6)(i)).

In your letter, you submitted relief request B21-3 for the 20 S/RVs at GGNS and requested relief from performing (1) Category B stroke and stroke time testing to the safety-related position at least once every 3 months in accordance with Paragraph IWV-3411 of the ASME Code and (2) remote position indication verification at least once every two years in accordance with IWV-3300 of the ASME Code. The basis for your request was Paragraph (a)(3)(ii) of 10 CFR 50.55a and your letter provided the basis that compliance with the ASME Code would result in a hardship or unusual difficulty without a compensating increase in the level of quality and safety.

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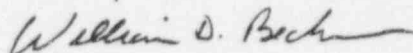
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Mr. C. Randy Hutchinson

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Enclosed is the staff's safety evaluation of relief request B21-3 for GGNS. Based on this evaluation, the staff has determined that you have demonstrated that compliance with the applicable ASME Code testing requirements would result in a hardship or unusual difficulty without a compensating increase in the level of quality and safety. Therefore, in accordance with Paragraph (a)(3)(ii) of 10 CFR 50.55a, the Commission hereby authorizes the proposed alternative to the ASME Code for GGNS as described in relief request B21-3 and, thus, relief request B21-3 is approved for use at GGNS.

Sincerely,



William D. Beckner, Director
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure: Safety Evaluation

cc w/encl: See next page

Mr. C. Randy Hutchinson

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Enclosed is the staff's safety evaluation of relief request B21-3 for GGNS. Based on this evaluation, the staff has determined that you have demonstrated that compliance with the applicable ASME Code testing requirements would result in a hardship or unusual difficulty without a compensating increase in the level of quality and safety. Therefore, in accordance with Paragraph (a)(3)(ii) of 10 CFR 50.55a, the Commission hereby authorizes the proposed alternative to the ASME Code for GGNS as described in relief request B21-3 and, thus, relief request B21-3 is approved for use at GGNS.

Sincerely,

ORIGINAL SIGNED BY:

William D. Beckner, Director
Project Directorate IV-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure: Safety Evaluation

cc w/encl: See next page

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Document Name: GG95403.REL

*See previous concurrence

OFC	(A)LA/PD4-1	PM/PD4-1	OGC*	D/PD4-1 <i>WDB</i>
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DATE	11/15/96	11/15/96	11/07/96	11/14/96
COPY	YES/NO	YES/NO	YES/NO	YES/NO

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Grand Gulf Nuclear Station

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