

FERMI 2

CORE OPERATING LIMITS REPORT

CYCLE 6

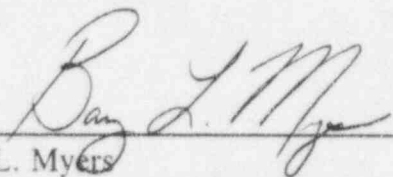
<i>Information and Procedures</i>				
DSN COLR Cycle 6	Revision 0	DCR # N/A	DTC TMTRM	File # 1754.01
IP Code	Date Approved 11/11/96	Released By	Date Issued	Recipient

FERMI 2

CORE OPERATING LIMITS REPORT

CYCLE 6

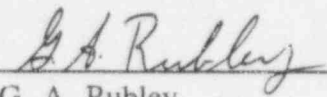
Prepared by:


B. L. Myers
Principal Engineer - Nuclear Fuel

11-6-96


Date

Reviewed by:


G. A. Rubley
Senior Engineer - Nuclear Fuel

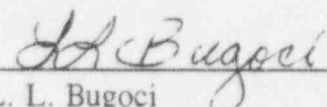
11-6-96

Date


J. M. Thorson
Supervisor - Reactor Engineering

11/6/96

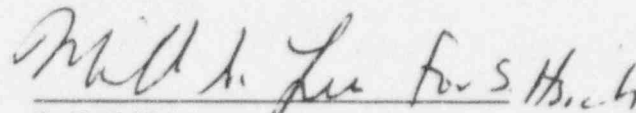
Date


L. L. Bugoci
COLR Checklist Reviewer

11/6/96

Date

Approved by:


S. T-C Hsieh
Supervisor - Nuclear Fuel

11/11/96

Date

NOVEMBER 1996

TABLE OF CONTENTS

1.0 INTRODUCTION AND SUMMARY	4
2.0 AVERAGE PLANAR LINEAR HEAT GENERATION RATE	5
2.1 Definition	5
2.2 Determination of MAPLHGR Limit	5
2.2.1 Calculation of MAPFAC(P)	7
2.2.2 Calculation of MAPFAC(F)	8
3.0 MINIMUM CRITICAL POWER RATIO	9
3.1 Definition	9
3.2 Determination of Operating Limit MCPR	9
3.3 Calculation of MCPR(P)	10
3.3.1 Calculation of K_p	11
3.4 Calculation of MCPR(F)	12
4.0 LINEAR HEAT GENERATION RATE	13
4.1 Definition	13
4.2 Determination of LHGR Limit	13
5.0 CONTROL ROD BLOCK INSTRUMENTATION	14
5.1 Definition	14
5.2 RBM Applicability	15
6.0 REFERENCES	16

LIST OF TABLES

TABLE 1	FUEL TYPE-DEPENDENT STANDARD MAPLHGR LIMITS	6
TABLE 2	FLOW-DEPENDENT MAPLHGR LIMIT COEFFICIENTS	8
TABLE 3	OLMCPR _{100/105} AS A FUNCTION OF EXPOSURE AND τ	10
TABLE 4	FLOW-DEPENDENT MCPR LIMIT COEFFICIENTS	12
TABLE 5	LHGR LIMITS FOR VARIOUS FUEL TYPES	13
TABLE 6	CONTROL ROD BLOCK INSTRUMENTATION SETPOINTS WITH FILTER	14

1.0 INTRODUCTION AND SUMMARY

This report provides the cycle specific plant operating limits, which are listed below, for Fermi 2, Cycle 6, as required by Technical Specifications 6.9.3. The analytical methods used to determine these core operating limits are those previously reviewed and approved by the Nuclear Regulatory Commission in GESTAR II.^{1,2,3,4,5}

The cycle specific limits contained within this report are valid for the full range of the licensed operating domain.^{6,7}

<u>OPERATING LIMIT</u>	<u>TECHNICAL SPECIFICATION</u>
APLHGR	3/4.2.1
MCPR	3/4.2.3
LHGR	3/4.2.4
RBM	3/4.3.6 & 3/4.1.4.3
APLHGR = AVERAGE PLANAR LINEAR HEAT GENERATION RATE	
MCPR = MINIMUM CRITICAL POWER RATIO	
LHGR = LINEAR HEAT GENERATION RATE	
RBM = ROD BLOCK MONITOR SETPOINTS	

2.0 AVERAGE PLANAR LINEAR HEAT GENERATION RATE

TECH SPEC IDENT	OPERATING LIMIT
3/4.2.1	APLHGR

2.1 Definition

The AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR) shall be applicable to a specific planar height and is equal to averaging the LINEAR HEAT GENERATION RATE over each fuel rod in the plane.

2.2 Determination of MAPLHGR Limit

The maximum APLHGR (MAPLHGR) limit is a function of reactor power, core flow, lattice type, and average planar exposure. The limit is developed to ensure gross cladding failure will not occur following a loss of coolant accident (LOCA) and that fuel thermal-mechanical design criteria will not be violated during any postulated transient events. The MAPLHGR limit ensures that the peak clad temperature during a LOCA will not exceed the limits as specified in 10CFR50.46(b)(1) and that the fuel design analysis criteria defined in References 1 and 2 will be met.

The MAPLHGR limit is calculated by the following equation:

$$MAPLHGR_{LIMIT} = MIN (MAPLHGR (P), MAPLHGR (F))$$

where:

$$MAPLHGR (P) = MAPFAC (P) \times MAPLHGR_{STD}$$

$$MAPLHGR (F) = MAPFAC (F) \times MAPLHGR_{STD}$$

MAPLHGR_{STD}, the standard MAPLHGR limit, is defined at a power of 3430 MWt and flow of 105 Mlbs/hr for each fuel type as a function of average planar exposure and is presented in Table 1. Since fuel types may contain more than one lattice type (axially), Table 1 represents the most limiting lattice type at each exposure point for that fuel type. When hand calculations are required as specified in Technical Specification 3/4.2.1, MAPLHGR_{STD} shall be determined by interpolation from Table 1.

MAPFAC(P), the core power-dependent MAPLHGR limit adjustment factor, shall be calculated by using Section 2.2.1.

MAPFAC(F), the core flow-dependent MAPLHGR limit adjustment factor, shall be calculated by using Section 2.2.2.

TABLE 1 FUEL TYPE-DEPENDENT STANDARD MAPLHGR LIMITS

Exposure GWD/ST	Standard MAPLHGR Limit (KW/FT)								
	Fuel Type								
	1	2	6	7	9	10	11	12	13
0.0			10.82	10.84	11.73	11.51	11.73	10.75	11.73
0.2	12.00	11.90	10.90	10.92	11.79	11.54	11.79	10.79	11.79
1.0	12.10	12.00	11.10	11.11	11.90	11.62	11.90	10.90	11.90
2.0			11.36	11.38	12.01	11.71	12.01	11.11	12.01
3.0			11.64	11.66	12.10	11.79	12.10	11.36	12.10
4.0			11.94	11.88	12.20	11.87	12.20	11.54	12.20
5.0	12.70	12.10	12.17	12.02	12.30	11.96	12.30	11.67	12.30
6.0			12.30	12.18	12.40	12.04	12.40	11.81	12.40
7.0			12.48	12.38	12.51	12.13	12.51	11.95	12.51
8.0			12.68	12.61	12.62	12.23	12.62	12.09	12.62
9.0			12.88	12.84	12.68	12.34	12.68	12.23	12.68
10.0	12.80	12.20	13.04	13.02	12.70	12.48	12.70	12.39	12.70
12.5			13.07	13.07	12.57	12.50	12.57	12.46	12.57
15.0	12.90	12.20	12.83	12.83	12.17	12.19	12.17	12.18	12.17
17.5					11.78	11.82	11.78	11.88	11.78
20.0	12.70	12.10	12.18	12.18	11.39	11.45	11.39	11.57	11.39
25.0	11.70	11.60	11.54	11.54	10.63	10.71	10.63	10.88	10.63
30.0	10.80	11.20			9.91	9.99	9.91	10.15	9.91
35.0			10.26	10.26	9.24	9.28	9.24	9.43	9.24
40.0	9.00	9.30			8.62	8.59	8.62	8.73	8.62
45.0			8.76	8.72	8.03	7.91	8.03	8.05	8.03
50.0					7.45	7.24	7.45	7.37	7.45
50.60				5.88					
50.80			5.88						
55.0					6.84	6.56	6.34	6.68	6.84

Fuel Types

1 = P8CIB176-4GZ-100M-150-T	10 = GE11-P9CUB353-10GZ-100M-146-T
2 = P8CIB219-4GZ-100M-150-T	11 = GE11-P9CUB331-11GZ-100M-146-T
6 = GE9B-P8CWB321-9GZ-80M-150-T	12 = GE11-P9CUB366-15GZ-100T-146-T
7 = GE9B-P8CWB321-10GZ-80M-150-T	13 = GE11-P9CUB331-11GZ-100M-146-T
9 = GE11-P9CUB331-11GZ-100M-146-T	

2.2.1 Calculation of MAPFAC(P)

The core power-dependent MAPLHGR limit adjustment factor, MAPFAC(P), shall be calculated by one of the following equations:

For $0 \leq P < 25$:

No thermal limits monitoring is required.

For $25 \leq P < 30$:

With turbine bypass OPERABLE,

For core flow ≤ 50 Mlbs/hr,

$$MAPFAC(P) = 0.606 + 0.0038 (P - 30)$$

For core flow > 50 Mlbs/hr,

$$MAPFAC(P) = 0.586 + 0.0038 (P - 30)$$

With turbine bypass INOPERABLE,

For core flow ≤ 50 Mlbs/hr,

$$MAPFAC(P) = 0.490 + 0.0050 (P - 30)$$

For core flow > 50 Mlbs/hr,

$$MAPFAC(P) = 0.438 + 0.0050 (P - 30)$$

For $30 \leq P \leq 100$:

$$MAPFAC(P) = 1.0 + 0.005224 (P - 100)$$

where: P = Core power (fraction of rated power times 100).

2.2.2 Calculation of MAPFAC(F)

The core flow-dependent MAPLHGR limit adjustment factor, MAPFAC(F), shall be calculated by the following equation:

$$MAPFAC(F) = MIN(1.0, A_F \times \frac{WT}{100} + B_F)$$

where:

WT = Core flow (Mlbs/hr).

A_F = Given in Table 2.

B_F = Given in Table 2.

TABLE 2 FLOW-DEPENDENT MAPLHGR LIMIT COEFFICIENTS

Maximum Core Flow* (Mlbs/hr)	A _F	B _F
110	0.6800	0.4340

*As limited by the Recirculation System MG Set mechanical scoop tube stop setting.

3.0 MINIMUM CRITICAL POWER RATIO

TECH SPEC IDENT	OPERATING LIMIT
3/4.2.3	MCPR

3.1 Definition

The CRITICAL POWER RATIO (CPR) shall be the ratio of that power in the assembly which is calculated by application of an NRC approved critical power correlation to cause some point in the assembly to experience boiling transition, divided by the actual assembly operating power.

The MINIMUM CRITICAL POWER RATIO (MCPR) shall be the smallest CPR that exists in the core.

3.2 Determination of Operating Limit MCPR

The required Operating Limit MCPR (OLMCPR) at steady-state rated power and flow operating conditions is derived from the established fuel cladding integrity Safety Limit MCPR of 1.09 and an analysis of abnormal operational transients. To ensure that the Safety Limit MCPR is not exceeded during any anticipated abnormal operational transient, the most limiting transients have been analyzed to determine which event will cause the largest reduction in CPR. Two different core average exposure conditions are evaluated. The result is an Operating Limit MCPR which is a function of exposure and τ . τ is a measure of scram speed, and is defined in Technical Specification Section 3/4.2.3.

The OLMCPR shall be calculated by the following equation:

$$OLMCPR = MAX (MCPR (P), MCPR (F))$$

MCPR(P), the core power-dependent MCPR operating limit, shall be calculated using Section 3.3.

MCPR(F), the core flow-dependent MCPR operating limit, shall be calculated using Section 3.4.

In case of **Single Loop Operation**, the Safety Limit MCPR is increased by 0.02, but OLMCPR does not change.

3.3 Calculation of MCPR(P)

MCPR(P), the core power-dependent MCPR operating limit, shall be calculated by the following equation:

$$MCPR(P) = K_p \times OLMCPR_{100/105}$$

OLMCPR_{100/105} shall be determined by interpolation from Table 3, and τ shall be calculated by using Technical Specification Section 3/4.2.3.

K_p, the core power-dependent MCPR Operating Limit adjustment factor, shall be calculated by using Section 3.3.1.

TABLE 3 OLMCPR_{100/105} AS A FUNCTION OF EXPOSURE AND τ

<u>CONDITION</u>	<u>EXPOSURE</u> <u>(MWD/ST)</u>		<u>OLMCPR_{100/105}*</u>
Both Turbine Bypass and Moisture Separator Reheater OPERABLE	BOC to 8500	$\tau = 0$	1.28
		$\tau = 1$	1.33
	8500 to EOC	$\tau = 0$	1.32
		$\tau = 1$	1.40
Either Turbine Bypass or Moisture Separator Reheater INOPERABLE	BOC to EOC	$\tau = 0$	1.36
		$\tau = 1$	1.44
Both Turbine Bypass and Moisture Separator Reheater INOPERABLE	BOC to EOC	$\tau = 0$	1.39
		$\tau = 1$	1.47

*The OLMCPR values reported here are for GE11 fuel and bound the GE6 and GE9B fuel types.

3.3.1 Calculation of K_p

The core power-dependent MCPR operating limit adjustment factor, K_p , shall be calculated by using one of the following equations:

For $0 \leq P < 25$:

No thermal limits monitoring is required.

For $25 \leq P < 30$:

When turbine bypass is OPERABLE,

$$K_p = \frac{(K_{BYP} + (0.026 \times (30 - P))) \times (1.09 / 1.07)}{OLMCPR_{100/105}}$$

where: $K_{BYP} = 1.90$ for core flow ≤ 50 Mlbs/hr
 $= 2.23$ for core flow > 50 Mlbs/hr

When turbine bypass is INOPERABLE,

$$K_p = \frac{(K_{BYP} + (0.054 \times (30 - P))) \times (1.09 / 1.07)}{OLMCPR_{100/105}}$$

where: $K_{BYP} = 2.26$ for core flow ≤ 50 Mlbs/hr
 $= 3.03$ for core flow > 50 Mlbs/hr

For $30 \leq P < 45$:

$$K_p = 1.28 + (0.0134 \times (45 - P))$$

For $45 \leq P < 60$:

$$K_p = 1.15 + (0.00867 \times (60 - P))$$

For $60 \leq P \leq 100$:

$$K_p = 1.0 + (0.00375 \times (100 - P))$$

where: P = Core power (fraction of rated power times 100).

3.4 Calculation of MCPR(F)

MCPR(F), the core flow-dependent MCPR operating limit, shall be calculated by using one of the following equations:

For $WT < 40$:

$$MCPR(F) = (1.09/1.07) \times (A_F \times \frac{WT}{100} + B_F) \times (1.0 + 0.0032 \times (40 - WT))$$

For $WT \geq 40$:

$$MCPR(F) = (1.09/1.07) \times MAX(1.20, (A_F \times \frac{WT}{100} + B_F))$$

where:

- WT = Core flow (Mlbs/hr).
- A_F = Given in Table 4.
- B_F = Given in Table 4.

TABLE 4 FLOW-DEPENDENT MCPR LIMIT COEFFICIENTS

Maximum Core Flow* (Mlbs/hr)	A _F	B _F
110	-0.600	1.731

*As limited by the Recirculation System MG Set mechanical scoop tube stop setting.

4.0 LINEAR HEAT GENERATION RATE

TECH SPEC IDENT	OPERATING LIMIT
3/4.2.4	LHGR

4.1 Definition

The LINEAR HEAT GENERATION RATE (LHGR) shall be the heat generation per unit length of fuel rod. It is the integral of the heat flux over the heat transfer area associated with the unit length.

4.2 Determination of LHGR Limit

The thermal expansion rates of UO₂ pellets and Zircaloy cladding are different in that, during heatup, the fuel pellet could come into contact with the cladding and create stress. By maintaining the operating LHGR below the limits stated in Table 5 and the operating MAPLHGR below those stated in Section 2.0, it is assured that all thermal-mechanical design bases and licensing limits for the fuel will be satisfied.

TABLE 5 LHGR LIMITS FOR VARIOUS FUEL TYPES

FUEL TYPE	LHGR LIMIT
P8CIB176-4GZ-100M-150-T	13.4 KW/FT
P8CIB219-4GZ-100M-150-T	13.4 KW/FT
GE9B-P8CWB321-9GZ-80M-150-T	14.4 KW/FT
GE9B-P8CWB321-10GZ-80M-150-T	14.4 KW/FT
GE11-P9CUB331-11GZ-100M-146-T	14.4 KW/FT
GE11-P9CUB353-10GZ-100M-146-T	14.4 KW/FT
GE11-P9CUB366-15GZ-100T-146-T	14.4 KW/FT

5.0 CONTROL ROD BLOCK INSTRUMENTATION

TECH SPEC IDENT	SETPOINT
3/4.3.6 & 3/4.1.4.3	RBM

5.1 Definition

The nominal trip setpoints and allowable values of the control rod withdrawal block instrumentation for use in Technical Specification 3/4.3.6 are shown in Table 6. These values are consistent with the bases of the APRM Rod Block Technical Specification Improvement Program (ARTS) and the MCPR operating limits.

TABLE 6 CONTROL ROD BLOCK INSTRUMENTATION SETPOINTS WITH FILTER

Setpoint	Trip Setpoint	Allowable Value
LPSP	27.0	28.6
IPSP	62.0	63.6
HPSP	82.0	83.6
LTSP	117.0	118.8
ITSP	112.2	114.0
HTSP	107.2	109
DTSP	94.0	92.3

where:

LPSP Low power setpoint; Rod Block Monitor (RBM) System trip automatically bypassed below this level

IPSP Intermediate power setpoint

HPSP High power setpoint

LTSP Low trip setpoint

ITSP Intermediate trip setpoint

HTSP High trip setpoint

DTSP Downscale trip setpoint

5.2 RBM Applicability

Technical Specifications 3/4.1.4.3 and 3/4.3.6 describe the conditions under which the Rod Block Monitor System must be OPERABLE. In addition to these requirements, at least one RBM channel must be OPERABLE when moving control rods with THERMAL POWER greater than or equal to 30% of RATED THERMAL POWER in order to protect for mechanical overpower limits.

6.0 REFERENCES

1. "General Electric Standard Application for Reactor Fuel (GESTAR II)," NEDE-24011-P-A, Revision 13
2. "The GESTR-LOCA and SAFER Models for the Evaluation of the Loss-of-Coolant Accident - SAFER/GESTR Application Methodology," NEDE 23785-1-PA, Revision 1, October 1984
3. "Fermi-2 SAFER/GESTR-LOCA, Loss-of-Coolant Accident Analysis," NEDC-31982P, July 1991, and Errata and Addenda No. 1, April 1992
4. "Lattice-Dependent MAPLHGR Report for Fermi Power Plant Unit 2 Reload 5 Cycle 6," GE Nuclear Energy, J11-02923MAPL, Revision 0, November 1996
5. "Supplemental Reload Licensing Report for Fermi Power Plant Unit 2 Reload 5, Cycle 6," GE Nuclear Energy, J11-02923SRLR, Revision 0, November 1996
6. Letter from T. G. Colburn to W. S. Orser, "Fermi-2 - Amendment No. 87 to Facility Operating License No. NPF-43 (TAC NO. M82102)," September 9, 1992
7. Letter from J. F. Stang to W. S. Orser, "Amendment No. 53 to Facility Operating License No. NPF-43: (TAC No. 69074)," July 27, 1990
8. "Maximum Extended Operating Domain Analysis for Detroit Edison Company Enrico Fermi Energy Center Unit 2," GE Nuclear Energy, NEDC-31843P, July 1990
9. Letter from R. J. Howard to M. K. Deora and H. L. Hubeny, "Operating Flow Dependent MCPR and MAPLHGR Thermal Limits," TDEC-PE-134, September 30, 1990
10. "Power Range Neutron Monitoring System," J. L. Leong, DC-4608, Vol. IX DCD, Rev. 0, September 29, 1992 and DC-4608 Vol. I Rev. D
11. Letter from B. R. Fischer to B. L. Myers, "Fermi-2, Cycle 6 Replacement of Leaker Bundle YJ2624," LB#262-96-167, October 14, 1996
12. Letter from R. J. Bragg to B. L. Myers, "Fermi-2 Cycle 6 Rod Withdrawal Error Analysis," RJB:96-27, November 1, 1996
13. Letter from R. J. Bragg to B. L. Myers, "ARTS Multipliers Update for Fermi 2 Cycle 6," RJB:96-29, November 1, 1996
14. Letter from C. J. Papandrea to Dr. Simon Hsieh, "Fermi 2 Cycle 6 Safety Limit MCPR Results," CJP2:96-163, August 23, 1996

6.0 REFERENCES (cont'd.)

15. Letter from R. J. Bragg to B. L. Myers, "Fermi- 2 Cycle SLMCPR Licensing Clarification," RJB:96-22, Rev. 1, October 28, 1996
16. Letter from B. R. Fischer to B. L. Myers, "Fermi 2 Cycle 6 SLMCPR Licensing Clarification - GE Proprietary Information," LB#262-96-159, October 8, 1996
17. Letter from R. J. Bragg to B. L. Myers, "Detroit Edison Questions Regarding the Fermi- 2, Cycle 6 Reload Licensing Report," RJB:96-34, November 5, 1996
18. Letter from Andrew J. Kugler (USNRC) to Douglas R. Gipson (Detroit Edison), "Fermi-2 - Issuance of Amendment RE: Cycle-Specific Safety Limit Minimum Critical Power Ratios for Cycle 6 (TAC NO. M96373), dated November 5, 1996