



**DUKE POWER**

DATE: November 7, 1996

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: McGuire Nuclear Station, Unit 1 and 2  
Docket No. 50-369  
Licensee Event Report 369/96-04, Revision 0  
Problem Investigation Process No.: 1-M96-2885

Gentlemen:

Pursuant to 10 CFR 50.73 Sections (a) (1) and (d), attached is Licensee Event Report 369/96-04, Revision 0, concerning incorrectly calibrated Reactor Coolant Loop Loss of Flow Bistables. This report is being submitted in accordance with 10 CFR 50.73 (a)(2)(i). This event is considered to be of no significance with respect to the health and safety of the public.

Very truly yours,

T.C. McMeekin

JWP/bcb

Attachment

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9611190104 961107  
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**LICENSEE EVENT REPORT (LER)**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

McGuire Nuclear Station Unit 1

DOCKET NUMBER (2)

05000369

PAGE (3)

1 of 1

TITLE (4)

Incorrectly Calibrated Reactor Coolant Loop Loss Of Flow Reactor Trip Bistables Due To A Procedural Technical Inaccuracy.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER(S)
10	10	96	96	004	0	11	07	96	N/A	05000
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR (Check one or more of the following) (11)										
OPERATING MODE (9)		1		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)
POWER LEVEL (10)		100		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
				20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
				20.405(a)(1)(iii)		X 50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
				20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
				20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME

J. W. Pitesa

TELEPHONE NUMBER

AREA CODE

(704)

875-4788

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
				NO					

SUPPLEMENTAL REPORT EXPECTED (14)

X YES (If yes, complete EXPECTED SUBMISSION DATE)

NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR
12	6	96

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

Unit Status: Unit 1 - Mode 1 (Power Operation) at 100 percent power.

**Event Description:**

On October 10, 1996, at approximately 2110, plant personnel determined that setpoints specified in maintenance calibration procedures used to adjust the Reactor trip bistables associated with the 1A Reactor Coolant Flow channels II and III did not comply with the values specified in Technical Specification (T/S) Table 2.2.1

**Event Cause:**

This event was caused by a procedural Technical Inaccuracy. A Root Cause Analysis is currently in progress.

**Corrective Action:**

Immediate: All affected procedures were changed. Instruments which were calibrated outside of allowable T/S limits were adjusted to within T/S limits.

Planned: Planned corrective actions are currently being evaluated.