



Boston Edison

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier

Vice President Nuclear Operations
and Station Director

November 14, 1996

BECO Ltr. #96-097

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

OCTOBER 1996 MONTHLY REPORT

In accordance with Pilgrim Nuclear Power Station Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report, please contact me directly.


L. J. Olivier

RLC/dmc/9458

Attachment

cc: Mr. Hubert Miller
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

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OPERATING DATA REPORT

DOCKET NO. 50-293
NAME: Pilgrim
COMPLETED BY: R. L. Cannon
TELEPHONE: (508) 830-8321
REPORT MONTH October 1996

OPERATING STATUS

NOTES

1. Unit Name Pilgrim I
2. Reporting Period October 1996
3. Licensed Thermal Power (MWt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:

No Changes

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	<u>50-293</u>
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REPORT MONTH	<u>October 1996</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	251	17	638
2	555	18	648
3	655	19	515
4	629	20	660
5	649	21	658
6	655	22	658
7	654	23	656
8	655	24	658
9	644	25	657
10	650	26	615
11	647	27	657
12	643	28	658
13	647	29	658
14	646	30	659
15	643	31	658
16	642		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATIONAL SUMMARY

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The unit started the period in power ascension at approximately 40 percent core thermal power (CTP). At 0139 hours on October 3, 1996, 100 percent CTP was achieved. At 1820 hours on October 4, 1996 power was reduced to approximately 80 percent CTP to perform a control rod pattern change. The plant was returned to 100 percent CTP at 0512 hours on October 5, 1996. One hundred percent CTP was maintained until 0201 hours on October 19, 1996, when power was reduced to approximately 50 percent CTP to accommodate a backwash of the main condenser. Following the backwash power was returned to 100% CTP at 2230 hours on October 19, 1996, where it maintained until 1646 hours on October 21, 1996. Power was reduced approximately 10 percent first by control rods and then by recirculation flow for operator training. On October 26, 1996, at 0200 hours reactor power was reduced to approximately 90 percent CTP to perform control rod exercising. Reactor power was returned to 100 percent CTP at 1859 hours on October 26, 1996, where it remained for the duration of the operating period.

SAFETY RELIEF VALVE CHALLENGES

MONTH OF OCTOBER 1996

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

DOCKET NO.	<u>50-293</u>
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REPORT MONTH	<u>October 1996</u>

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: February 1, 1997.
3. Scheduled date for restart following next refueling: March 14, 1997.
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1995 refueling outage (RFO-10) is of a different design than that loaded in the previous refueling outage and consists of 136 new fuel assemblies.
7.
 - (a) There are 580 fuel assemblies in the core.
 - (b) There are 1765 fuel assemblies in the spent fuel pool.
 - (c) There are 148 fuel assemblies on-site awaiting receipt inspection for RFO-11.
8.
 - (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2891 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.
 - (b) The planned spent fuel storage capacity is 3859 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1103 fuel assemblies.

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

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SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
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No major safety related corrective maintenance was completed during this reporting period.

UNIT SHUTDOWNS AND POWER REDUCTIONS

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NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
25	10/04/96	S	0.0	B	N/A	N/A	N/A	N/A	Power reduction to 80% CTP perform rod pattern change.
26	10/19/96	S	0.0	B	N/A	N/A	N/A	N/A	Power reduction to 50% CTP to perform a backwash of main condenser.

1
F-Forced
S-Sched

2
A-Equip Failure
B-Main or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training
& License Examination
F-Admin
G-Operator Error
H-Other

3
1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

4&5
Exhibit F & H
Instructions for
Preparations of
Data Entry Sheet
Licensee Event Report
(LER) File (NUREG-1022)