

November 13, 1996

Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, WI 53201

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SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON CONTROL ROOM HABITABILITY
EVALUATION TO SUPPORT STEAM GENERATOR REPLACEMENT - POINT BEACH
NUCLEAR PLANT, UNITS 1 AND 2 (TAC NOS. M96697, M96698, M96741 and
M96742)

Dear Mr. Link:

As discussed with your staff on October 18, 1996, the Nuclear Regulatory Commission noted that your submittal of August 5, 1996, included a change in the loss-of-coolant accident doses as described in Final Safety Analysis Report Section 14.2.4, "Radiological Consequences of a Steam Generator Tube Rupture Accident." To support an independent assessment of dose consequences as a result of changes in your plant design, the staff determined that additional information is required. Please respond to Part A of the enclosed request for additional information (RAI) by November 20, 1996, and Part B of the RAI by December 2, 1996. If you have any questions, please contact me at (301) 415-1380.

Sincerely,

Original signed by
Linda L. Gundrum, Project Manager
Project Directorate III-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

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Docket Nos. 50-266
and 50-301

Enclosure: RAI

cc w/encl: See next page
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DATE	11/12/96		11/13/96		11/13/96	

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

November 13, 1996

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Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, WI 53201

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Sincerely,

A handwritten signature in cursive script, reading "Linda L. Gundrum".

Linda L. Gundrum, Project Manager
Project Directorate III-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-266
and 50-301

Enclosure: RAI

cc w/encl: See next page

Mr. Robert E. Link, Vice President
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Unit Nos. 1 and 2

cc:

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Chairman
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Ms. Sarah Jenkins
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Public Service Commission of Wisconsin
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Madison, Wisconsin 53707-7854

REQUEST FOR ADDITIONAL INFORMATION ON CONTROL ROOM HABITABILITY
EVALUATION TO SUPPORT STEAM GENERATOR REPLACEMENT
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

Part A

1. Describe the modes of operation of the control room heating, ventilation, and air conditioning (HVAC) system, including the following: control room habitable volume in cubic feet, range of assumed flow rates (recirculated flow, fresh air inlet flow, filtered air flow) used for dose calculations, assumptions of inleakage and outleakage, pressurization assumed in inches of water gage and basis for assumption, descriptions of the filters used including assumed removal capabilities. Describe any additional protective measures included in calculation of control room dose.
2. Define each postulated source term resulting from a control rod drop accident, steam generator tube rupture accident (SGTR), loss of reactor coolant flow accident (locked rotor), and main steam line break (MSLB). Define the receptor points used for the control room, exclusion area boundary (EAB), and low population zone (LPZ). Provide the following information for each source/receptor pair (plan view and cross-sectional figures may be helpful in describing the source, building configuration, and intake relationships):
 - a. height of physical top of vent or stack or other assumed release point,
 - b. dimensions of buildings and other structures adjacent to or separating release point and intake/receptor,
 - c. horizontal straight line distance between source and receptor.
 - d. height of physical intake or receptor,
 - e. direction (in degrees) from intake to source, as follows: assuming one is standing at the intake looking toward the source, looking toward the North is 360°, toward the East is 90°, toward the South is 180°, etc. This grid system must be congruent with the meteorological data collection grid system requested in Question 4.
 - f. An hourly listing of 5 years of onsite meteorological data representative of long-term conditions at the site. Data should be provided in a form consistent with the format described in Appendix A to section 2.3.3, "Onsite Meteorological Measurements Programs," of NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants, LWR Edition." Although reference is made to a magnetic tape, data provided on other electronic media (e.g., a 3 1/2 inch disk) will also be acceptable.
 - g. Describe the mode of operation of the control room HVAC for each accident for the 30-day control room operator dose calculations.

ENCLOSURE

3. What are the basis and assumptions for the X/Qs used in your past and current control room habitability assessments?
4. What is the basis for the atmospheric dispersion estimates used in the dose assessment for the EAB, LPZ, and any other off-site locations? For each of the 16 compass directions, what is the minimum distance from each assumed release point to the site boundary or to the shoreline for those cases in which the site boundary extends past the shoreline?

Part B

Analyze the following accidents: (1) steam generator tube rupture, (2) control rod ejection, (3) locked rotor, and (4) main steamline break. Submit a copy of your accident analysis report that contains the calculated control room, EAB and LPZ doses and a comparison of the results to the 10 CFR Parts 50 and 100 acceptance criteria. Also provide sufficient documentation of your analyses to support an independent evaluation, such as the code inputs, the references for the code inputs (e.g., UFSAR or new docketed analyses), and the dose calculations.