

November 13, 1996

Mr. Robert E. Link, Vice President
Nuclear Power Department
Wisconsin Electric Power Company
231 West Michigan Street, Room P379
Milwaukee, WI 53201

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SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON TECHNICAL SPECIFICATION
CHANGE REQUEST 192 - POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2
(TAC NOS. M96741 and M96742)

Dear Mr. Link:

On September 30, 1996, Change Request 192 was submitted. This change requests changes to Technical Specification (TS) 15.3.3, "Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray," TS 15.3.7, "Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray," and TS 15.5.2, "Containment." As discussed with your staff on November 8, 1996, the staff determined that additional information is required. Please respond to the enclosed request for additional information (RAI) by November 26, 1996. If you have any questions, please contact me at (301) 415-1380.

Sincerely,

Linda L. Gundrum

Linda L. Gundrum, Project Manager
Project Directorate III-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-266
and 50-301

Enclosure: RAI

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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A handwritten signature in cursive script, reading "Linda L. Gundrum".

Linda L. Gundrum, Project Manager
Project Directorate III-1
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

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and 50-301

Enclosure: RAI

cc w/enc1: See next page

Mr. Robert E. Link, Vice President
Wisconsin Electric Power Company

Point Beach Nuclear Plant
Unit Nos. 1 and 2

cc:

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REQUEST FOR ADDITIONAL INFORMATION ON CHANGE REQUEST 192
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

The following questions relate to Attachment 3 of Wisconsin Electric Power Company's September 30, 1996, application to amend Technical Specification (TS) 15.3.3, "Emergency Core Cooling System, Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray," TS 15.3.7, "Auxiliary Cooling Systems, Air Recirculation Fan Coolers, and Containment Spray," and TS 15.5.2, "Containment."

1. Attachment 3 indicates that Westinghouse performed a new post-LOCA containment pressure and temperature analysis in 1996. Do the mass and energy boundary conditions and the containment response model for the 1996 analysis reflect any and all post-Operating License plant modifications that might affect the containment pressure and temperature response to a loss-of-coolant accident? Will the 1996 analysis be incorporated into the Final Safety Analysis Report as the new analysis of record?
2. Have secondary system breaks reflecting Bulletin 80-04, "Analysis of a Pressurized Water Reactor Main Steam Line Break with Containment Feedwater Addition," 10 CFR 50.49 limitations, and any applicable plant modifications been considered in the new analyses?
3. In switching to use of American Nuclear Society Standard 5.1-1979 decay heat, has a two-sigma uncertainty margin been added?
4. Explain how the 25 percent air cooler capacity reduction is implemented in the containment response model (e.g., reduction in overall heat transfer coefficient: reduction in water flow?). Explain why the method properly reflects the new service water system conditions.
5. One of the changes described in Attachment 3 is the use of a value of 33°F for "injection water cooling by the residual heat removal (RHR) heat exchanger." Please clarify the use of the 33°F value. (Does the containment model not continuously compute the injection water conditions using a simplified primary system model?)
6. Please explain reliance on the RHR and component cooling water (CCW) systems for containment heat removal (when are the systems placed in service as assumed in the analysis.) Please describe the CCW system's ability to withstand a single failure based on current TS.

ENCLOSURE