

ENCLOSURE 1

NOTICE OF VIOLATION

Duke Power Company
McGuire Nuclear Station

Docket Nos. 50-369 and 50-370
License Nos. NPF-9 and NPF-17

The following violations were identified during an inspection conducted on June 21 - July 20, 1985. The Severity Levels were assigned in accordance with the NRC Enforcement Policy (10 CFR Part 2, Appendix C).

1. Technical Specification (TS) 4.9.2 requires, for mode 6 operation, that each source range neutron flux monitor be demonstrated operable by the performance of an analog channel operational test within 8 hours prior to initial core alterations and at least once per 7 days.

TS 4.0.4 declares that entry into an operational mode shall not be made unless the surveillance requirement associated with the limiting condition for operation have been performed within the stated surveillance interval.

Contrary to the above,

- a. On May 1, 1985, Unit 1 entered mode 6 without performing an analog channel operation test on the source range monitors within the previous seven days.
- b. On May 6, 1985, Unit 1 core alterations were begun without performing an analog channel operational test on the source range monitors within the previous eight hours.

This is a Severity Level IV violation (Supplement I).

2. Technical Specification 6.8.1.a requires that current written approved procedures be established, implemented and maintained covering those surveillance tests required by Technical Specifications. Implicit in the provisions of these requirements is the requirement that the procedures embody sufficient detail to facilitate the successful accomplishment of the task.

Contrary to the above:

- a. On July 12, 1985, during the performance of a manual reactor trip surveillance test, the applicable procedure, PT-2-A-4600-56, Manual Reactor Trip Functional Test, was not followed in that the feedwater isolation reset switches were not depressed while tripping reactor trip breaker as required. This resulted in an inadvertent feedwater isolation.

- b. On July 12, 1985, during the performance of test PT-0-A-4601-08A, Solid State Protection System Train A, the procedure was inadequate in detail in that it did not specify the deactivation of the P-4 permissive prior to closing the reactor trip breaker. This resulted in an inadvertent feedwater isolation.
- c. On May 6, 1985, a shift supervisor erroneously signed step 6.5 on Enclosure 13.1 of Unit 1 procedure MP-1-A-7150-41, Control Rod Drive Shaft Latching and Unlatching, which stated that applicable surveillance requirements of TS 3.9.2 has been met. The surveillance had not been performed. This in turn led to a mode change without completing prerequisites required by TS 4.0.4

This is a Severity Level IV violation (Supplement I).

- 3. 10 CFR 50, Appendix B, Criteria XVI as implemented by Duke Power Company Topical Report, Quality Assurance Program Duke-1-A, Amendment 7, Section 17.2.16 requires that conditions adverse to quality be promptly identified and corrected.

Contrary to the above, on May 22, 1985, a QC inspector discovered a violation of the electrical separation of certain Unit 2 safety related cable yet no corrective action was taken until May 28, 1985.

This is a Severity Level IV violation (Supplement I).

Pursuant to 10 CFR 2.201, you are required to submit to this office within 30 days of the date of this Notice, a written statement or explanation in reply, including: (1) admission or denial of the alleged violations; (2) the reasons for the violations if admitted; (3) the corrective steps which have been taken and the results achieved; (4) corrective steps which will be taken to avoid further violations; and (5) the date when full compliance will be achieved.

Security or safeguards information should be submitted as an enclosure to facilitate withholding it from public disclosure as required by 10 CFR 2.790(d) or 10 CFR 73.21.

AUG 14 1985

Date: _____