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August 23, 1985

United States Nuclear Regulatory Commission
Washington, DC 20555

ATTENTION: Mr. George W. Knighton, Chief
Licensing Branch 3
Office of Nuclear Reactor Regulation

SUBJECT: Beaver Valley Power Station - Unit No. 2
Docket No. 50-412
Elimination of Arbitrary Intermediate Pipe Breaks

REFERENCES: 1) DLC letter 2NRC-5-042, dated March 12, 1985
2) NRC letter to DLC dated May 21, 1985

Gentlemen:

In Reference 1, Duquesne Light Company (DLC) requested approval to eliminate arbitrary intermediate pipe breaks in high energy piping systems. The response in Reference 2 concluded that the request was found to be acceptable.

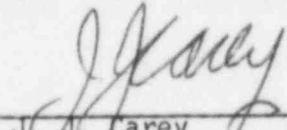
The purpose of this letter is to clarify two points as follows:

1. For ASME Code Class 1 piping, the stress criterion for postulating intermediate breaks is 3.0 Sm instead of 2.4 Sm. Reference 1 refers to the criteria of Branch Technical Positions ASB 3-1 and MEB-1; however, the change to the 3.0 Sm value was made in FSAR Section 3.6B.2.1.1.1 in Amendment 8, dated September 1984. Further clarification was also recently given in FSAR Table 1.9-2 on SRP Section No. 3.6.2, Amendment 10, dated May 1985.
2. As described in Reference 1, arbitrary intermediate breaks are being eliminated in certain Non-Nuclear Safety (NNS) systems. Although the enclosure to Reference 2 does not specifically address NNS systems, from the cover letter, it is our understanding that the acceptance applies to all the specific systems identified in the request, including NNS systems.

The foregoing will be reflected in a future FSAR amendment that will revise Section 3.6.

DUQUESNE LIGHT COMPANY

By


J. J. Carey
Vice President

JJS/wjs

cc: Mr. B. K. Singh, Project Manager
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