



**Consumers  
Power**  
**POWERING  
MICHIGAN'S PROGRESS**

Patrick M Donnelly  
Plant Manager

Big Rock Point Nuclear Plant, 10269 US-31 North, Charlevoix, MI 49720

November 1, 1996

Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

Enclosed please find the statistical data for the Big Rock Point Nuclear Plant covering the period of October 1, 1996 through October 31 1996.

Sincerely,

P M Donnelly  
Plant Manager

**Enclosures**

cc: Administrator Region III, Nuclear Regulatory Commission  
DRHahn, Department of Public Health  
JRPadgett, Michigan Public Service Commission  
Raben, Michigan Department of Labor  
MPCass, American Nuclear Insures  
FYost, Research Services Utility Data Institute  
GLHayes, General Electric Company  
RAFenech, Palisades  
GCWithrow, Big Rock Point  
NRC Resident Inspector, Big Rock Point  
Document Control, Big Rock Point, 740/22\*35\*10  
File

9611140065 961031  
PDR ADOCK 05000155  
R PDR

140008

11  
1024

Operating Status Report - Nuclear Regulatory Commission

Prepared by: James R. Johnston Date: November 1, 1996  
 Reviewed by: J. Mervish Date: 11/6/96  
 (Reactor Engineer or Alternate)

Copy to be sent to the following by the seventh day of each month:

Nuclear Regulatory Commission  
 Document Control Desk  
 Washington, DC 20555

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 Lisle, IL 60532-4351

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 3500 North Logan  
 Lansing, MI 48906

JR Padgett  
 Michigan Public Service Commission  
 PO Box 30221  
 Lansing, MI 48909

Robert Aben, Assistant Chief  
 Bureau of Construction Codes  
 Michigan Department of Labor  
 7150 Harris Drive, Box 30015  
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American Nuclear Insurers  
 Att: MP Cass  
 Town Center, Suite 300S  
 29 South Main Street  
 West Hartford, CT 06107-2445

GL Hayes  
 General Electric  
 2311 West 22 St.  
 Suite 201  
 Oak Brook, IL 60540

Routing Copy:

RA Fenech, Palisades Plant  
 SM McIntyre, P26-204  
 GC Withrow, Big Rock Point Plant  
 NRC Resident Inspector, Big Rock Point Plant  
 Document Control Center, Big Rock Point Plant  
 DCC 740\*22\*35\*10  
 (740\*22\*10\*04) Cross Reference

Fred Yost  
 Director, Research Services  
 Utility Data Institute  
 1700 K Street, N.W.  
 Suite 400  
 Washington, DC 20006



# NUCLEAR OPERATIONS DEPARTMENT Unit Shutdowns and Power Reductions

Report Month October, 1996	Docket Number 55-150	Unit Big Rock Point Plant	Date November 1, 1996	Completed by JR Johnston	Telephone (616) 547-8223
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Number	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report Number	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause and Corrective Action To Prevent Recurrence
96-05	10/10/96	F	39.7 Hrs	A	4	-----	-----	-----	On 10/10/96 at 13:20; Power was reduced to allow repairs on #1 Reactor Feedwater pump (packing gland leak). After repairs were completed, the unit was returned to normal power operation.
96-06	10/23/96 thru 10/31/96	F	199.7 Hrs	H	4	-----	-----	-----	On 10/23/96 at 17:17; The unit was removed from service before manually scrambling the reactor to fully insert the remaining withdrawn control rod drives (planned). The Electrical Equipment Qualifications (EEQ) concerns was the reason for the outage. When the EEQ concerns are resolved, the unit will be returned to service.

<sup>1</sup>F = Forced  
S = Scheduled

<sup>2</sup>Reason:  
A = Equipment Failure (Explain)  
B = Maintenance of Test  
C = Refueling  
D = Regulatory Restriction  
E = Operator Training and License Examination  
F = Administrative  
G = Operational Error (Explain)  
H = Other (Explain)

<sup>3</sup>Method:  
1 = Manual  
2 = Manual Scram  
3 = Automatic Scram  
4 = Other (Explain)

<sup>4</sup>Exhibit G = Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>Exhibit I = Same Source

## GREYBOOK OPERATING DATA REPORT

DOCKET NO. 50-155

DATE: 11 / 1 / 96

BY: JR JOHNSTON

PHONE: 616-547-8223

## OPERATING STATUS

## NOTES:

1. UNIT NAME: BIG ROCK POINT NUCLEAR PLANT
2. REPORTING PERIOD: 10 / 96
3. LICENSED THERMAL POWER (MW): 240
4. NAMEPLATE RATING (GROSS MWE): 75
5. DESIGN ELECTRICAL RATING (NET MWE): 72
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 71.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 67.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THRU 7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):
10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	745.0	7320.0	294499.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	545.3	4815.3	214851.3
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	545.3	4758.3	211561.8
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	121080.0	1033119.0	40992190.0
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	38281.0	327273.0	13028255.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	36376.6	310812.7	12326750.7
19. UNIT SERVICE FACTOR	73.2%	65.0%	71.8%
20. UNIT AVAILABILITY FACTOR	73.2%	65.0%	71.8%
21. UNIT CAPACITY FACTOR (USING MDC NET)	72.9%	63.4%	62.2%
22. UNIT CAPACITY FACTOR (USING DER NET)	67.8%	59.0%	58.1%
23. UNIT FORCED OUTAGE RATE	26.8%	7.0%	10.6%

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, & DURATION OF EACH): REFUELING, APRIL, 1996, 60 DAYS.

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOVEMBER 3, 1996.

DAY AVERAGE DAILY POWER(MWT) (MWEN) (10/96) - CYCLE 29

1	224.71	67.79
2	226.67	68.32
3	228.25	68.76
4	227.29	68.41
5	225.92	68.12
6	227.46	68.50
7	226.50	68.24
8	228.04	68.69
9	226.75	68.28
10	189.75	56.92
11	175.88	53.00
12	223.46	67.20
13	224.17	67.42
14	223.04	67.05
15	228.04	68.81
16	227.21	68.03
17	227.63	69.14
18	227.21	68.29
19	227.42	68.60
20	228.13	68.59
21	226.79	68.54
22	229.13	69.07
23	145.58	39.92
24	0.00	0.00
25	0.00	0.00
26	0.00	0.00
27	0.00	0.00
28	0.00	0.00
29	0.00	0.00
30	0.00	0.00
31	0.00	0.00

Refueling Information Request

1. Facility Name: Big Rock Point Plant
2. Scheduled date for next refueling shutdown: April, 1997.
3. Scheduled date for restart following shutdown: 60 Day Duration.
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No

If yes, explain:

If no, has the reload fuel design and core configuration been reviewed by Plant Safety Review Committee to determine whether any unreviewed safety questions as associated with the core reload (Reference 10 CFR, Section 50.59)? To be performed once core loading determined for cycle 30.

If no review has taken place, when is it scheduled?

5. Scheduled date(s) for submittal of proposed licensing action and supporting information: N/A
6. Important licensing considerations associated with refueling, eg. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design new operating procedures: None
7. Number of fuel assemblies in: core 84; spent fuel pool storage 358; new fuel storage 0.
8. Present licensed spent fuel pool storage capacity: 441  
Size of any increase in licensed storage capacity that has been requested or planned (in number of fuel assemblies): 0
9. Projected date of last refueling that can be discharged to spent fuel pool assuming the present license capacity: Last total core off load 1996; Last refueling 1999.