

WOLF CREEK GENERATING STATION

MONTHLY OPERATING REPORT

MONTH: July YEAR: 1985

Docket No.: STN 50-482

Facility Operating License No.: NPF-42

Report No. 5

Submitted by:

Kansas Gas and Electric Company

The following report highlights the operating experience of Wolf Creek Generating Station for the month of July, 1985. This report is being provided pursuant to Technical Specification 6.9.1.8.

I. SUMMARY OF OPERATING EXPERIENCE

During the month of July, Power Ascension Testing was completed at the fifty percent and seventy-five percent power plateaus. This testing included the Large Load Reduction Test from a reactor power level of seventy-five percent, which was completed satisfactorily without a reactor trip.

Several Engineered Safety Features actuations and Reactor trips occurred due to feedwater control problems. These problems were a combination of personnel activities and equipment problems, and have been resolved by combination of equipment re-adjustments, procedure modifications, and operator retraining.

II. MAJOR SAFETY RELATED MAINTENANCE ACTIVITIES

The major safety related maintenance activities that were performed during the month of July involved the Main Steamline Isolation Valves (MSLIV's) and Feedwater Isolation Valves (FWIV's). These activities included the development of a comprehensive trouble-shooting procedure, the construction of a test bench for testing the 4-way valves on the MSLIV's and FWIV's, and the repair of oil leaks.

Other activities included adjusting the pressure equalizing valve on the containment access hatch, repair of the jacket water heaters on the standby emergency diesel generators, and repair of hinge pin leakage on a Main Feedwater check valve.

III. CHANGES, TESTS, AND EXPERIMENTS

The following is a brief description of safety evaluations performed pursuant to 10 CFR 50.59 on changes, tests, and experiments during the month of July.

1. Temporary Modification 85-SE-55 - Replace pipe caps with quick disconnect couplings on two test connections on the personnel airlock door seals to simplify the attachment of equipment to leak test the seals. No unreviewed safety or environmental questions are generated as a result of this temporary change.
2. Temporary Modification 85-SE-56 - Install a blank plate at an existing joint in duct 200-5NL-4 to cut off normal exhaust flow from the Fuel Building. No unreviewed safety or environmental questions are generated as a result of this temporary change.
3. Temporary Modification 85-SE-57 - Install temporary insulation on check valve BB-V8378B, normal charging connection to the Reactor Coolant System, and on flow element BB-FE-427, measuring the combined flows of the Loop 2 hot and cold leg RTD manifolds. Insulation will remain until the next major outage. No unreviewed safety or environmental questions are generated as a result of this temporary change.

4. Temporary Modification 85-SE-58 - Install scaffolding to work on damper GL-D-172 on the 2047' elevation of the Auxiliary Building. No unreviewed safety or environmental questions are generated as a result of this temporary change.
5. Temporary Modification 85-SE-59 - Install scaffolding to reach ducts on suction and discharge of fans CGF03A and B, main steam enclosure exhaust fans, in Area 4 of the Auxiliary Building on elevation 2047'. No unreviewed safety or environmental questions are generated as a result of this temporary change.
6. Temporary Modification 85-SE-60 - Install scaffolding on elevation 1974' of the Auxiliary Building to clean and insulate various 4" lines in the Essential Service Water and Auxiliary Building Heating, Ventilation, and Air Conditioning systems to room coolers SGL09A and B, SGL10A and B, SGL12A and B, and SGL13 A and B. Scaffolding was to be installed in only one of two redundant safety equipment rooms at one time. No unreviewed safety or environmental questions are generated as a result of this temporary change.
7. Temporary Modification 85-SE-61 - Install scaffolding in Area 5 of the Auxiliary Building on the 2039' elevation to facilitate cleaning of miscellaneous heating, ventilation and air conditioning equipment. No unreviewed safety or environmental questions are generated as a result of this temporary change.
8. Temporary Modification 85-SE-62 - Remove covers from Essential Service Water valve pits to pump out the pits. Work was to be performed in one bay at a time, leaving the cover in place on the other bay. Covers were to be replaced if severe weather threatened, or if work was to be suspended for a period of time. No unreviewed safety or environmental questions are generated as a result of this temporary change.
9. Temporary Modification 85-SE-63 - Install scaffolding on the 2026' elevation of the Auxiliary Building to work on damper GF-D-065. No unreviewed safety or environmental questions are generated as a result of this temporary change.
10. Temporary Modification 85-SE-64 - Install scaffolding on the 2026' elevation of the Auxiliary Building to repair a void in the concrete. No unreviewed safety or environmental questions are generated as a result of this temporary change.
11. Temporary Modification 85-SE-65 - Install scaffolding at the west end of wall A in the north Residual Heat Removal pump room of the Auxiliary Building on the 1967' elevation. No unreviewed safety or environmental questions are generated as a result of this temporary change.
12. Temporary Modification 85-SE-66 - Install scaffolding in Area 5 on the 2047' elevation of the Auxiliary Building to install insulation on Loop 3 Main Steamline Isolation Valve. No unreviewed safety or environmental questions are generated as a result of this temporary change.

13. Temporary Modification 85-SE-67 - Install scaffolding in Room 1323 (North Piping Penetration Room) on the 2000' elevation of the Auxiliary Building to clean and insulate Essential Service Water system piping. No unreviewed safety or environmental questions are generated as a result of this temporary change.
14. Temporary Modification 85-SE-68 - Install a blank flange in pipe EC 070-HCD-4, fuel pool cleanup system, in the Refueling Water Storage Tank valve pit to facilitate repair of valve BN-V002, at the inlet to the tank. No unreviewed safety or environmental questions are generated as a result of this temporary change.
15. Temporary Modification 85-SE-69 - Reduction of the Control Room air conditioning flow rate to document noise reduction while maintaining temperature at or below 84 degrees F per Technical Specification 4.7.12. No unreviewed safety or environmental questions are generated as a result of this temporary change.
16. Temporary Modification 85-SE-70 - Add a 4" connection to the flange between valves AP-V005 and AP-LV-2, condensate storage tank isolation and level control valves, to supply demineralized water to the condensate storage tank, TAP01, on the normal makeup line for the tank. No unreviewed safety or environmental questions are generated as a result of this temporary change.
17. Temporary Modification 85-SE-71 - Install a temporary nitrogen supply to tanks TKA 02, 03, 04 and 05, auxiliary feedwater control/main steam atmospheric relief valves accumulators, until pump INT01P returned to normal operation. No unreviewed safety or environmental questions are generated as a result of this temporary change.
18. Temporary Modification 85-SE-72 - Install scaffolding in Room 1322 (South Piping Penetration Room) on the 2000' elevation of the Auxiliary Building to clean and insulate Essential Service Water system piping. No unreviewed safety or environmental questions are generated as a result of this temporary change.
19. Temporary Modification 85-SE-73 - Install a variable speed sheeve on SGK04, Main Control Room air conditioning unit, to support Control Room Air Conditioning flow reduction. No unreviewed safety or environmental questions are generated as a result of this temporary change.
20. Temporary Modification 85-SE-74 - Install a jumper line to supply nitrogen to safety injection accumulator "D", bypassing valve EP-HV-8875D, nitrogen supply valve, until this valve is restored to normal service. The jumper will run from EP-V058, test connection on the nitrogen header, to EP-V103, drain valve on the level transmitter. No unreviewed safety or environmental questions are generated as a result of this temporary change.

21. Temporary Modification 85-SE-75 - Install scaffolding to high point vent valves EM-V156 and EM-V242 in the high pressure coolant injection system thus facilitating performance of a surveillance test. No unreviewed safety or environmental questions are generated as a result of this temporary change.
22. Temporary Modification 85-SE-76 - Install pressure instrumentation on the tube side of High Pressure Feedwater heaters at existing pressure points AE-V052, AE-V055, AE-V058 and AE-V062. No unreviewed safety or environmental questions are generated as a result of this temporary change.
23. Temporary Modification 85-SE-77 - Install scaffolding at Location TA and T8 (acid neutralizer and degasifier area) and Location TA and T3 (railroad bay) on 2000' elevation of Turbine Building. No unreviewed safety or environmental questions are generated as a result of this temporary change.
24. Temporary Modification 85-SE-78 - Replace bolts in bonnets of valves AE-V006, suction isolation for Main Feedwater Pump "B", and AE-V009, suction isolation for Main Feedwater Pump "A", to allow sealant application and stop bonnet leaks. No unreviewed safety or environmental questions are generated as a result of this temporary change.

OPERATING DATA REPORT

DOCKET NO. SIN 50-482
 WOLF CREEK GENERATING STATION
 KANSAS GAS AND ELECTRIC COMPANY
 DATE 8-01-85
 COMPLETED BY M. Williams
 TELEPHONE 316-364-8831

OPERATING STATUS

1. Reporting Period: July, 1985 Gross Hours in Reporting Period: 744
 2. Currently Authorized Power Level (MWt): 3411 Max. Depend. Capacity (MWe-Net): 1117
 Design Electrical Rating (MWe-Net): 1170
 3. Power Level to Which Restricted (If Any) (Mwe-Net): N/A
 4. Reasons for restriction (If Any): N/A
- | | This Month | Yr to Date | Cumulative |
|---|------------------|------------------|------------------|
| 5. Number of Hours Reactor was Critical | <u>542</u> | <u>1180</u> | <u>1180</u> |
| 6. Reactor Reserve Shutdown Hours | <u>202</u> | <u>443</u> | <u>443</u> |
| 7. Hours Generator on Line | <u>504</u> | <u>809</u> | <u>809</u> |
| 8. Unit Reserve Shutdown Hours | <u>240</u> | <u>343</u> | <u>343</u> |
| 9. Gross Thermal Energy Generated (MWH) | <u>1,025,780</u> | <u>1,382,456</u> | <u>1,382,456</u> |
| 10. Gross Electrical Energy Generated (MWH) | <u>333,736</u> | <u>413,344</u> | <u>413,344</u> |
| 11. Net Electrical Energy Generated (MWH) | <u>300,007</u> | <u>347,919</u> | <u>347,919</u> |
| 12. Reactor Service Factor | <u>N/A</u> | | |
| 13. Reactor Availability Factor | <u>N/A</u> | | |
| 14. Unit Service Factor | <u>N/A</u> | | |
| 15. Unit Availability Factor | <u>N/A</u> | | |
| 16. Unit Capacity Factor (Using MDC) | <u>N/A</u> | | |
| 17. Unit Capacity Factor (Using Design MWe) | <u>N/A</u> | | |
| 18. Unit Forced Outage Rate | <u>N/A</u> | | |
19. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of each): None
 20. If Shut Down at End of Report Period, Estimated Date of Startup: 8-1-85
 21. Units in test Status (Prior to Commercial Operation):

Forecast	Achieved
Initial Criticality	<u>5-22-85</u>
Initial Electricity	<u>6-13-85</u>
Commercial Operation	<u>9-09-85</u>

MONTH July, 1985

DAY AVERAGE DAILY POWER LEVEL		DAY AVERAGE DAILY POWER LEVEL	
(MWe-Net)		(MWe-Net)	
1	<u>0</u>	17	<u>356</u>
2	<u>0</u>	18	<u>476</u>
3	<u>0</u>	19	<u>511</u>
4	<u>0</u>	20	<u>643</u>
5	<u>0</u>	21	<u>721</u>
6	<u>423</u>	22	<u>782</u>
7	<u>474</u>	23	<u>809</u>
8	<u>481</u>	24	<u>225</u>
9	<u>0</u>	25	<u>765</u>
10	<u>0</u>	26	<u>804</u>
11	<u>0</u>	27	<u>806</u>
12	<u>0</u>	28	<u>704</u>
13	<u>9</u>	29	<u>801</u>
14	<u>441</u>	30	<u>832</u>
15	<u>467</u>	31	<u>719</u>
16	<u>291</u>		

INSTRUCTIONS:

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

UNIT SHUTDOWN AND POWER REDUCTIONS

DOCKET NO. SIN 50-482

WOLF CREEK GENERATING STATION

KANSAS GAS AND ELECTRIC COMPANY

DATE 8-01-85COMPLETED BY M. WilliamsTELEPHONE 316-364-8831REPORT MONTH July, 1985

No	Date	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON (1)	METHODS SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
3	850629	S	107.3	B	4	Manual Trip per Startup Test Program from Auxiliary Shutdown Panel to demonstrate its operability.
4	850709	F	90.9	A	3	Trip due to feedwater control problem during Startup Test Program transient testing. A test recorder induced a false control signal. (Licensee Event Reports 85-049, 85-050, and 85-042).
5	850715	S	11.3	B	2	Reactor trip to test negative rate trip circuitry and analyze plant performance per Power Ascension Testing.
6	850723	F	10.7	A	3	Trip due to instrument power supply failure resulting in loss of control power to a feedpump. (Licensee Event Report 85-054)
7	850728	S	0	B	4	Large Load Reduction Test per Power Ascension Testing.
8	850731	F	20.2	A	3	Trip due to failure of a power range Nuclear Instrumentation channel while a second channel was under test. (Licensee Event Report 85-058)

SUMMARY: During the month of July, Power Ascension Testing was completed at the fifty and seventy-five percent power plateaus. This testing included a large load reduction test and Reactor trip tests. Several trips occurred due to feedwater system related problems.

The Gross Electrical Energy Generated and Net Electrical Energy Generated figures for July have been adjusted due to recalibration on the kilowatt-hour meters.

(1) REASON:

- A: EQUIPMENT FAILURE (EXPLAIN)
- B: MAINTENANCE OR TEST
- C: REFUELING
- D: REGULATORY RESTRICTION
- E: OPERATOR TRAINING AND LICENSE EXAMINATION
- F: ADMINISTRATIVE
- G: OPERATIONAL ERROR (EXPLAIN)
- H: OTHER (EXPLAIN)

(2) METHOD:

- 1. MANUAL
- 2. MANUAL SCRAM
- 3. AUTOMATIC SCRAM
- 4. OTHER (EXPLAIN)

KANSAS GAS AND ELECTRIC COMPANY

WOLF CREEK GENERATING STATION

UNIT NO. 1

MONTH July, 1985

SUMMARY OF OPERATING EXPERIENCE

Listed below in chronological sequence is a summary of operating experiences for this month which required load reduction or resulted in significant non-load related incidents.

<u>DATE</u>	<u>TIME</u>	<u>EVENT</u>
June 6, 1985	2250	Erroneously reported that a Reactor Trip and Safety Injection occurred in Monthly Report #4. This entry should read: Reactor Trip, Auxiliary Feed Actuation Signal (AFAS) and Feedwater Isolation Signal (FWIS) occurred.
July 1, 1985	0000	Plant in Mode 3.
July 2, 1985	2001	Received a Control Room Ventilation Isolation Signal (CRVIS) due to the spurious spiking of the gaseous channel of Control Room radiation monitor GK-RE-04. Reportable per 10 CFR 50.72 and 50.73.
July 4, 1985	2338	Commenced Reactor startup. Plant entered Mode 2.
July 5, 1985	0045	Plant entered Mode 3 due to operator discretion.
	0501	Commenced Reactor startup. Plant entered Mode 2.
	0804	Entered Mode 1.
	0946	AFAS and FWIS occurred due to high-high level in Steam Generator "A". Reportable per 10 CFR 50.72 and 50.73.
	0947	Plant entered Mode 2.
	1016	Plant entered Mode 1.
	1119	Synchronized with grid.
	1506	Received CRVIS due to spurious spiking of GK-RE-04. Reportable per 10 CFR 50.72 and 50.73.
July 7, 1985	0550	Received CRVIS due to the spurious spiking of GK-RE-04. Reportable per 10 CFR 50.72 and 50.73.

<u>DATE</u>	<u>TIME</u>	<u>EVENT</u>
July 8, 1985	0216	Received CRVIS due to the spurious spiking of GK-RE-04. Reportable per 10 CFR 50.72 and 50.73.
	2205	Received CRVIS due to the spurious spiking of GK-RE-04. Reportable per 10 CFR 50.72 and 50.73.
July 9, 1985	0932	Received CRVIS due to the spurious spiking of GK-RE-04. Reportable per 10 CFR 50.72 and 50.73.
	1115	Reactor Trip, AFAS, and FWIS occurred on low-low steam generator level signal due to a personnel error. Entered Mode 3. Reportable per 10 CFR 50.72 and 50.73.
July 10, 1985	0228	Commenced Reactor startup. Entered Mode 2.
	0547	Received FWIS due to high level in Steam Generator "B". Reportable per 10 CFR 50.72 and 50.73.
	0715	Plant entered Mode 1.
	0815	Received FWIS due to high level in Steam Generator "B". Reportable per 10 CFR 50.72 and 50.73.
	0817	Reactor trip occurred due to low Steam Generator level. Entered Mode 3. Reportable per 10 CFR 50.72 and 50.73.
	1156	Commenced Reactor startup. Entered Mode 2.
	1321	Received FWIS due to high level in Steam Generator "B". Reportable per 10 CFR 50.72 and 50.73.
	2347	Entered Mode 1.
July 11, 1985	0220	Received FWIS and AFAS due to high level in Steam Generator "A". Reportable per 10 CFR 50.72 and 50.73.
	0221	Reactor trip occurred due to high flux on the Intermediate Range Nuclear Instrumentation, due to overly conservative setpoints. Entered Mode 3. Reportable per 10 CFR 50.72 and 50.73.
July 12, 1985	0445	Received CRVIS due to broken detector tape on a Control Room chlorine monitor. Reportable per 10 CFR 50.72 and 50.73.
	0736	Received CRVIS due to the spurious spiking of GK-RE-04. Reportable per 10 CFR 50.72 and 50.73.

<u>DATE</u>	<u>TIME</u>	<u>EVENT</u>
	1246	Received CRVIS due to the spurious spiking of GK-RE-04. Reportable per 10 CFR 50.72 and 50.73.
	2158	Commenced Reactor startup. Entered Mode 2.
July 13, 1985	0212	Entered Mode 1.
	0607	Synchronized with grid.
July 15, 1985	2245	Reactor trip initiated per Power Ascension Testing. Entered Mode 3.
July 16, 1985	0236	Commenced Reactor startup. Entered Mode 2.
	0326	Reentered Mode 3 due to operator discretion.
	0526	Commenced Reactor startup. Entered Mode 2.
	0848	Entered Mode 1.
	1005	Synchronized with grid.
July 17, 1985	0848	Received CRVIS due to spurious signal from Control Room Radiation monitor due to a faulty bypass switch. Reportable per 10 CFR 50.72 and 50.73.
July 22, 1985	0353	Received CRVIS due to broken detector tape on Control Room chlorine monitor. Reportable per 10 CFR 50.72 and 50.73.
July 23, 1985	0807	Reactor trip, FWIS and AFAS occurred due to loss of control power to Main Feedwater Pump. Entered Mode 3. Reportable per 10 CFR 50.72 and 50.73.
	1356	Commenced Reactor startup. Entered Mode 2.
	1727	Entered Mode 1.
	1855	Synchronized with grid.
July 28, 1985	0059	CRVIS occurred due to a spurious Control Room chlorine monitor spike. Reportable per 10 CFR 50.72 and 50.73.
	0615	Completed Large Load Reduction Test from seventy-five percent reactor power. Reactor stable at twenty-five percent power.
July 31, 1985	0348	Reactor trip occurred due to a Nuclear Instrumentation channel spike while a second channel was out of service. Entered Mode 3. Reportable per 10 CFR 50.72 and 50.73.



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

August 15, 1985

Director, Office of Resource Management
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. R.P. Denise, Director
Division of Reactor Safety and Projects
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-198
Re: Docket No. STN 50-482
Subj: July, 1985 Monthly Operating Report

Gentlemen:

Enclosed is the July, 1985 Monthly Operating Report for Wolf Creek Generating Station. This submittal is being made in accordance with the requirements of Technical Specification 6.9.1.8.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

Glenn L. Koester
Vice President, Nuclear

CLK:dab

Enclosure

xc: PO'Connor (2), w/a
JTaylor (12), w/a
JCummins, w/a

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