



Northeast
Utilities System

107 Selden Street, Berlin, CT 06037

Northeast Utilities Service Company
P.O. Box 270
Hartford, CT 06141-0270
(860) 665-5000

November 7, 1996

Docket No. 50-336
B15995

Re: 10 CFR 50.73(a)(2)(ii)(B)

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

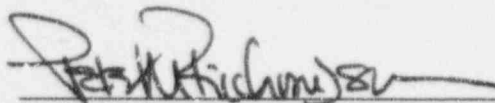
This letter forwards Licensee Event Report (LER) 96-032-00, documenting an event that occurred at Millstone Nuclear Power Station, Unit No. 2 on October 8, 1996. This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(ii)(B).

The following are NNECO's commitments made within this letter. All other statements made within this letter are for information only.

- B15995-01: The conditions involved in this event are still under investigation. As a result of the potential for unanalyzed water hammer, an evaluation of the operation and design of the RCS vent system is being performed. Based on this evaluation, appropriate modifications and/or procedure changes will be implemented to ensure that allowable pipe stresses will not be exceeded during system operation.
- B15995-02: A supplemental LER will be issued to provide the results of the investigation including the cause of the event and resultant corrective actions.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY



P. M. Richardson
Director - Millstone Unit No. 2

IE221

120041

cc: see page 2

9611130220 961107
PDR ADOCK 05000336
S PDR

Attachment: LER 96-032-00

cc: H. J. Miller, Region I Administrator
D. P. Beaulieu, Resident Inspector, Millstone Unit No. 2
D. G. McDonald, Jr., NRC Project Manager, Millstone Unit No. 2

LICENSEE EVENT REPORT (LER)(See reverse for required number of
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY
INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS
LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED
BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN
ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (IT-
6 #33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC
20565-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104),
OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Millstone Nuclear Power Station Unit 2

DOCKET NUMBER (2)

05000336

PAGE (3)

1 of 3

TITLE (4)

Potential Failure of Reactor Coolant System Vent System to Perform Design Function

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	08	96	96	-- 032 --	00	11	07	96	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)		5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11)							
POWER LEVEL (10)		000	20.2201(b)		20.2203(a)(2)(v)		50.73(a)(2)(i)		50.73(a)(2)(viii)	
			20.2203(a)(1)		20.2203(a)(3)(i)		<input checked="" type="checkbox"/> 50.73(a)(2)(ii)		50.73(a)(2)(x)	
			20.2203(a)(2)(i)		20.2203(a)(3)(ii)		50.73(a)(2)(iii)		73.71	
			20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		OTHER	
			20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		Specify in Abstract below or in NRC Form 366A	
			20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)			

LICENSEE CONTACT FOR THIS LER (12)

NAME

R. T. Laudenat, MP2 Nuclear Licensing Manager

TELEPHONE NUMBER (Include Area Code)

(860) 444-5248

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE).	<input type="checkbox"/> NO	EXPECTED SUBMISSION	MONTH	DAY	YEAR
			03	14	97

ABSTRACT (Limit to 1400 spaces, i.e., approximately 10 double-spaced typewritten lines) (16)

On October 8, 1996, several conditions were identified that could result in the reactor coolant system (RCS) vent system not being able to perform its design function. The RCS vent system is used to vent noncondensable gases from the RCS during post accident conditions. This system was installed to comply with NUREG-0737, "Clarification of TMI Action Plan Requirements," Item II.B.1, "Reactor Coolant System Vents." A review of the operation and design of the system has identified conditions that could prevent the ability of the RCS vent system to perform its function. These conditions include the potential for water hammer transients during system operation that could result in exceeding piping allowable stresses.

The conditions involved in this event are still under investigation. As a result of the potential for unanalyzed water hammer, an evaluation of the operation and design of the RCS vent system is being performed. Based on this evaluation, appropriate modifications and/or procedure changes will be implemented to ensure that allowable pipe stresses will not be exceeded during system operation.

A supplemental LER will be issued to provide the results of the investigation including the cause of the event and resultant corrective actions.

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER		REVISION NUMBER	
		96	--	032	--	00
Millstone Nuclear Power Station Unit 2	05000336					2 of 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

I. Description of Event

On October 8, 1996, several conditions were identified that could result in the reactor coolant system (RCS) vent system not being able to perform its design function. At the time of discovery of this event, the unit was in Mode 5 at 0 percent power.

The RCS vent system is used to vent noncondensable gases from the RCS during post accident conditions. This system was installed to comply with NUREG-0737, "Clarification of TMI Action Plan Requirements," Item II.B.1, "Reactor Coolant System Vents." A review of the operation and design of the system has identified conditions that could prevent the ability of the RCS vent system to perform its function. These conditions include the potential for water hammer transients during system operation that could result in exceeding piping allowable stresses. The investigation of this event is continuing.

This event is being reported in accordance with 10 CFR 50.73(a)(2)(ii)(B), as a condition that resulted in the nuclear power plant being outside the design basis of the plant. This event was reported in accordance with 10 CFR 50.72(b)(2)(i) on October 8, 1996.

II. Cause of Event

Based on the results of the continuing investigation, a supplemental LER will be issued detailing the identified cause of this event.

III. Analysis of Event

The RCS vent system provides the capability for removing noncondensable gases collected in the RCS to the containment atmosphere during post accident conditions. The RCS vent system utilizes two separate vent paths, one from the reactor vessel head and one from the pressurizer steam space which discharge to a common sparger inside containment. Each vent path includes a valve manifold with two sets of double isolation solenoid valves arranged in parallel. Unanalyzed water hammer transients could result in unacceptable loads on the piping and supports, potentially rupturing the lines. Depending on the location of the rupture, an unisolable release path from the RCS to the containment could occur. The safety significance of this event is still being evaluated based on the results of the investigation of this event.

IV. Corrective Action

The conditions involved in this event are still under investigation. As a result of the potential for unanalyzed water hammer, an evaluation of the operation and design of the RCS vent system is being performed. Based on this evaluation, appropriate modifications and/or procedure changes will be implemented to ensure that allowable pipe stresses will not be exceeded during system operation.

A supplemental LER will be issued to provide the results of the investigation including the cause of the event and resultant corrective actions.

V. Additional Information

None

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

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Millstone Nuclear Power Station Unit 2	05000336	YEAR	SEQUENTIAL NUMBER		REVISION NUMBER	3 of 3
		96	--	032	-- 00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Similar Events

None

Manufacturer Data

None