

PUBLIC SERVICE COMPANY OF COLORADO
FORT ST. VRAIN NUCLEAR GENERATING STATION

MONTHLY OPERATIONS REPORT

NO. 138

July, 1985

FORM 248 22 0218

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This report contains the highlights of the Fort St. Vrain, Unit No. 1, activities operated under the provisions of the Nuclear Regulatory Commission Operating License DPR-34. This report is for the month of July, 1985.

1.0 NARRATIVE SUMMARY OF OPERATING EXPERIENCE AND MAJOR SAFETY RELATED MAINTENANCE

The first part of the month was spent preparing the systems and procedures for start-up.

All interim procedures for Appendix R are in place, and training has been completed. New interim control rod drive Technical Specifications and surveillances are in the place.

The reactor was taken critical on July 20, 1985, at about 4:00 a.m. and remained there until approximately 12:00 midnight on July 23, 1985, when a high moisture scram occurred while trying to establish a shutdown seal on "A" circulator.

Both steam driven boiler feed pumps were placed in service and operated smoothly.

A system 46 chiller was established on "A" helium purification train, and enhanced moisture removal in the front end cooler was accomplished. This system will be established on "B" train once it is placed in service.

The digital valve installed on "1A" main drain experienced leakage problems. The original valve was reinstalled, and Engineering will conduct further testing on the digital valves for future use.

2.0 SINGLE RELEASES OF RADIOACTIVITY OR RADIATION EXPOSURE IN EXCESS OF 10% OF THE ALLOWABLE ANNUAL VALUE

None

3.0 INDICATION OF FAILED FUEL RESULTING FROM IRRADIATED FUEL EXAMINATIONS

None

4.0 MONTHLY OPERATING DATA REPORT

Attached

TECH
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OPERATING DATA REPORT

DOCKET NO. 50-267
DATE August 8, 1985
COMPLETED BY Frank Novachek
TELEPHONE (303) 785-2224

OPERATING STATUS

1. Unit Name: Fort St. Vrain
2. Reporting Period: 850701 through 850731
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None
9. Power Level To Which Restricted, If Any (Net MWe): 280
10. Reasons for Restrictions, If Any: Per commitment to NRC, long term operation above 85% power pending completion of B-0 Startup Testing.

NOTES

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	<u>744</u>	<u>5,087</u>	<u>53,352</u>
12. Number of Hours Reactor Was Critical	<u>67.7</u>	<u>67.7</u>	<u>27,219.1</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>0.0</u>	<u>18,468</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>109.5</u>	<u>109.5</u>	<u>9,861,823.9</u>
17. Gross Electrical Energy Generated (MWH)	<u>0.0</u>	<u>0.0</u>	<u>3,248,594</u>
18. Net Electrical Energy Generated (MWH)	<u>-2,973.0</u>	<u>-15,255.0</u>	<u>2,912,703</u>
19. Unit Service Factor	<u>0.0</u>	<u>0.0</u>	<u>34.6</u>
20. Unit Availability Factor	<u>0.0</u>	<u>0.0</u>	<u>34.6</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>0.0</u>	<u>16.6</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>0.0</u>	<u>16.6</u>
23. Unit Forced Outage Rate	<u>100.0</u>	<u>100.0</u>	<u>58.2</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):	<u>None</u>		

25. If Shut Down at End of Report Period, Estimated Date of Startup: September 1, 1985

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	<u>N/A</u>	<u>N/A</u>
INITIAL ELECTRICITY	<u>N/A</u>	<u>N/A</u>
COMMERCIAL OPERATION	<u>N/A</u>	<u>N/A</u>

AVERAGE DAILY UNIT POWER LEVEL

TSP-3
Attachment-3A
Issue 2
Page 1 of 1

Docket No. 50-267

Unit Fort St. Vrain

Date August 8, 1985

Completed By Frank Novachek

Telephone (303) 785-2224

Month July, 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>0.0</u>
2	<u>0.0</u>
3	<u>0.0</u>
4	<u>0.0</u>
5	<u>0.0</u>
6	<u>0.0</u>
7	<u>0.0</u>
8	<u>0.0</u>
9	<u>0.0</u>
10	<u>0.0</u>
11	<u>0.0</u>
12	<u>0.0</u>
13	<u>0.0</u>
14	<u>0.0</u>
15	<u>0.0</u>
16	<u>0.0</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>0.0</u>
18	<u>0.0</u>
19	<u>0.0</u>
20	<u>0.0</u>
21	<u>0.0</u>
22	<u>0.0</u>
23	<u>0.0</u>
24	<u>0.0</u>
25	<u>0.0</u>
26	<u>0.0</u>
27	<u>0.0</u>
28	<u>0.0</u>
29	<u>0.0</u>
30	<u>0.0</u>
31	<u>0.0</u>

*Generator on line but no net generation.

50-267

UNIT NAME Fort St. Vrain

DATE August 8, 1985

COMPLETED BY Frank Novachek

TELEPHONE (303) 785-2224

REPORT MONTH August, 1985

NO.	DATE	TYPE	DURATION	REASON	METHOD OF SHUTTING DOWN REACTOR	LER #	SYSTEM CODE	COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
84-006	840601	F	484.3	A	3	50-267/85-002	AB	BL0	Completion of Circulator Bolting Changeout and Primary Coolant cleanup
85-001	850720	F	192	G	3	50-267/85-012	AB	XXXXXX	Primary Coolant Cleanup

REFUELING INFORMATION

1. Name of Facility	Fort St. Vrain Unit No. 1
2. Scheduled date for next refueling shutdown.	January 1, 1987
3. Scheduled date for restart following refueling.	February 1, 1987
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?	No
If answer is yes, what, in general, will these be?	-----
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10 CFR Section 50.59)?	No
If no such review has taken place, when is it scheduled?	1985
5. Scheduled date(s) for submitting proposed licensing action and supporting information.	-----
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	-----
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.	a) 1482 HTGR fuel elements b) 137 spent fuel elements

REFUELING INFORMATION (CONTINUED)

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one-third of core (approximately 500 HTGR elements). No change is planned.
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	1992 under Agreements AT(04-3)-633 and DE-SC07-79ID01370 between Public Service Company of Colorado, and General Atomic Company, and DOE.*

- * The 1992 estimated date is based on the understanding that spent fuel discharged during the term of the Agreements will be stored by DOE at the Idaho Chemical Processing Plant. The storage capacity has evidently been sized to accommodate eight fuel segments. It is estimated that the eighth fuel segment will be discharged in 1992.



Public Service

Public Service
Company of Colorado

16805 WCR 19 1/2, Platteville, Colorado 80651

August 15, 1985
Fort St. Vrain
Unit No. 1
P-85287

Office of Inspection and Enforcement
ATTN: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket No. 50-267

SUBJECT: Monthly Operations
Report For July, 1985

REFERENCE: Facility Operating
License No. DPR-34

Dear Sir:

Enclosed, please find the Monthly Operations Report for the month of July, 1985.

Sincerely,

J. W. Gahm
Manager, Nuclear Production
Fort St. Vrain Nuclear
Generating Station

Enclosure

cc: Mr. Robert D. Martin, NRC

JWG:djm

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