

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
 UNIT Fort Calhoun Station
 DATE August 9, 1985
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

MONTH July, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>470.5</u>
2	<u>469.4</u>
3	<u>469.2</u>
4	<u>469.1</u>
5	<u>469.0</u>
6	<u>468.9</u>
7	<u>468.1</u>
8	<u>467.3</u>
9	<u>466.3</u>
10	<u>465.5</u>
11	<u>465.1</u>
12	<u>465.5</u>
13	<u>464.9</u>
14	<u>464.3</u>
15	<u>464.8</u>
16	<u>467.0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>467.8</u>
18	<u>467.6</u>
19	<u>466.9</u>
20	<u>466.9</u>
21	<u>467.0</u>
22	<u>467.4</u>
23	<u>467.8</u>
24	<u>467.0</u>
25	<u>466.2</u>
26	<u>466.0</u>
27	<u>466.2</u>
28	<u>466.6</u>
29	<u>467.5</u>
30	<u>467.4</u>
31	<u>470.0</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

8508210215 850731
 PDR ADOCK 05000285
 R PDR

IE24 1/1

OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE August 9, 1985
 COMPLETED BY T. P. Matthews
 TELEPHONE (402) 536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: July, 1985
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 502
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.0</u>	<u>5,087.0</u>	<u>103,873.0</u>
12. Number Of Hours Reactor Was Critical	<u>744.0</u>	<u>5,060.6</u>	<u>80,340.8</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1,309.5</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>5,052.7</u>	<u>79,720.1</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,110,686.9</u>	<u>7,478,935.4</u>	<u>101,665,702.4</u>
17. Gross Electrical Energy Generated (MWH)	<u>364,894.0</u>	<u>2,523,278.0</u>	<u>33,292,903.0</u>
18. Net Electrical Energy Generated (MWH)	<u>347,599.1</u>	<u>2,407,385.5</u>	<u>31,819,022.8</u>
19. Unit Service Factor	<u>100.0</u>	<u>99.3</u>	<u>76.7</u>
20. Unit Availability Factor	<u>100.0</u>	<u>99.3</u>	<u>76.7</u>
21. Unit Capacity Factor (Using MDC Net)	<u>97.7</u>	<u>99.0</u>	<u>66.6</u>
22. Unit Capacity Factor (Using DER Net)	<u>97.7</u>	<u>99.0</u>	<u>64.4</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>3.6</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

1985 Refueling Shutdown is tentatively scheduled for October, 1985 with startup in December, 1985.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation): N/A

Forecast

Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July, 1985

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE August 9, 1985
COMPLETED BY T. P. Matthews
TELEPHONE (402) 536-4733

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns during the month of July.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

⁵
Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending July, 1985.

1. Scheduled date for next refueling shutdown. October, 1985
2. Scheduled date for restart following refueling. December, 1985
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be?
Technical Specifications change to accommodate increased radial peaks due to further reduction in radial leakage.
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____
 - c. If no such review has taken place, when is it scheduled? _____
4. Scheduled date(s) for submitting proposed licensing action and support information. September, 1985
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Methodology Changes

June, 1985

6. The number of fuel assemblies:
 - a) in the core 133 assemblies
 - b) in the spent fuel pool 305 "
 - c) spent fuel pool storage capacity 729 "
 - d) planned spent fuel pool storage capacity May be increased via fuel pin consolidation "
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1996

Prepared by James J. Tasican for J. Rogers

Date August 2, 1985

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

July, 1985
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at a nominal 100% power throughout July, 1985. Four Reactor Operator and two Senior Reactor Operator candidates passed the NRC license examination. No candidates failed. Training programs continue for Equipment Operator-Nuclear (Turbine Room), emergency plan, C/RP Technicians and general employee training.

New fuel receipt and inspections of the 44 assemblies for Cycle 10 were completed in July. No abnormalities were noted during these inspections.

The annual emergency preparedness drill was held on July 10 and the exercise was held on July 24. There were no major deficiencies noted by the NRC exercise observers. The modified OSC/TSC organization was implemented and functioned well.

Responses to the INPO evaluation and assistance visit were prepared and submitted in July.

Reduction in long term low level radioactive waste continued during July. Several shipments of High Integrity Containers (HICs) aided in eliminating several drums of waste which have been stockpiled at the Station

No safety valve or PORV challenges or failures occurred.

A. PERFORMANCE CHARACTERISTICS

None

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

None

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
SP-FAUD-1	Fuel Assembly Uplift Condition Detection. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved the evaluation of data from a surveillance test to verify that a fuel assembly uplift condition did not exist.
SP-EEQ-1	LOCA Qualified Electrical Equipment Identification Check. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 because this procedure is simply an inventory list of qualified equipment and indications located on various panels and control boards in the plant. No plant operations or evolutions are involved.

System Acceptance Committee Packages for July, 1985:

<u>Package</u>	<u>Description/Analysis</u>
EEAR FC-84-73	Replacement of Containment Penetration Splices. This modification provided more reliable safety related cables/splices under accident and post-accident conditions. This modification has no adverse effect on the safety analysis.
EEAR FC-82-94	EHC Hydraulic Power Unit Modification. This modification was installed in order to prevent overpressurization of the recirculating tank (a portion of the transfer and fuller earth filtering unit of the EHC). This modification would protect the recirculating tank from overpressurization should a relief valve fail to open. The transfer and fuller earth filtering unit comes into existence when the EHC main reservoir is being filled. This modification has no adverse effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

During the month of July, the biannual leak rate surveillance test for the Containment Personnel Air Lock (PAL) was completed. The previous six month leak rate was 3,560 sccm. The July, 1985, leak rate for the PAL doors was found at 3,600 sccm.

This increases the previous "B" and "C" leak rate total of 9,427.53 sccm to 9,467.53 sccm. The current leak rate total is well below the allowed leakage of .6 La (62,951 sccm) as specified in 10CFR50 Appendix J. The next test that will affect the "B" and "C" leak rate total is the upcoming 1985 refueling outage leak rate test scheduled to start in September, 1985.

F. CHANGE IN PLANT OPERATING STAFF

During July Calvin Taylor, David Sweeney and John Borger started work as Auxiliary Operators-Nuclear. Dennis Schwieger started work in July as an Operations Helper. Dennis Bonsall, an Equipment Operator-Nuclear Licensed, resigned in July.

G. TRAINING

Four Reactor Operator and two Senior Reactor Operator candidates passed the NRC license examination. No candidates failed. Training programs continue for Equipment Operator-Nuclear (Turbine Room), emergency plan, C/RP Technicians and general employee training.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

None

II. MAINTENANCE (Significant Safety Related)

None

W. Gary Gates

W. Gary Gates
Manager
Fort Calhoun Station

Omaha Public Power District
1623 Harney Omaha, Nebraska 68102
402/536-4000

August 13, 1985
LIC-85-370

Mr. James M. Taylor, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

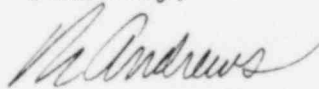
Reference: Docket No. 50-285

Dear Mr. Taylor:

July Monthly Operating Report

Please find enclosed ten (10) copies of the July, 1985 Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,



R. L. Andrews
Division Manager
Nuclear Production

RLA/TPM/dao

Enclosures

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Nuclear Safety Analysis Center
INPO Records Center
American Nuclear Insurers
NRC File

IE 24
1/2