



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PORTLAND GENERAL ELECTRIC COMPANY
EUGENE WATER AND ELECTRIC BOARD
PACIFIC POWER AND LIGHT COMPANY
DOCKET NO. 50-344
TROJAN NUCLEAR PLANT
AMENDMENT TO POSSESSION-ONLY LICENSE

Amendment No. 195
License No. NPF-1

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Portland General Electric Company, et al. (PGE or licensee) dated November 2, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will be maintained in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

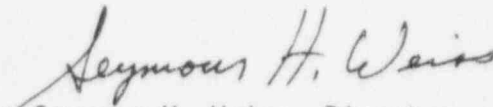
2. Accordingly, the license is amended by changes to the Appendix A Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-1 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 195, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. The licensee shall maintain the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Seymour H. Weiss, Director
Non-Power Reactors and Decommissioning
Project Directorate
Division of Reactor Program Management
Office of Nuclear Reactor Regulation

Attachment:
Appendix A Technical
Specifications

Date of Issuance: October 31, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 195

POSSESSION-ONLY LICENSE NO. NPF-1

DOCKET NO. 50-344

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

5-1
5-2
5-4
5-7
5-9
5-10
5-11
5-12
5-15
5-17
5-18

INSERT

5-1
5-2
5-4
5-7
5-9
5-10
5-11
5-12
5-15
5-17
5-18

5.0 ADMINISTRATIVE CONTROLS

5.1 Responsibility

- 5.1.1 The Trojan Site Executive and Plant General Manager (hereafter called the Plant General Manager), shall have overall responsibility for the facility and shall delegate in writing the succession to this responsibility during his absence.
- The Plant General Manager or his designee, in accordance with approved administrative procedures, shall approve prior to implementation, each proposed test or experiment related to the safe storage of irradiated fuel and proposed changes and modifications to structures, systems or equipment that affect the safe storage of irradiated fuel.
- 5.1.2 The Shift Manager shall be responsible for the operational command function. During any absence of the Shift Manager from the Control Room, another operator qualified to stand watch in the Control Room shall be designated to assume the command function. The individual maintaining the command function shall remain in the Control Room.
-

5.0 ADMINISTRATIVE CONTROLS

5.2 Organization

5.2.1 General Organizational Requirements

Facility and corporate organizations shall be established for the facility staff and corporate management, respectively. These organizations shall include the positions for activities affecting the safe storage of irradiated fuel.

- a. Lines of authority, responsibility, and communication shall be defined and established throughout highest management levels, intermediate levels, and all operating organization positions. These relationships shall be documented and updated, as appropriate, in organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation;
- b. The Plant General Manager shall have overall responsibility for the facility and shall have control over those facility activities necessary for operation and maintenance of structures and systems necessary for the safe storage of irradiated fuel;
- c. The Plant General Manager shall also have corporate responsibility for overall nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to ensure the safe storage of irradiated fuel; and
- d. The individuals who train the CERTIFIED FUEL HANDLERS, carry out radiation protection functions, or perform quality assurance functions may report to the appropriate line manager; however, these individuals shall have sufficient organizational freedom to ensure the ability to perform their assigned functions.

(continued)

5.2 Organization

5.2.2 Facility Staff (continued)

modifications (including decommissioning activities) the following guidelines shall be followed on a temporary basis:

1. An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time;
2. An individual should not be permitted to work more than 16 hours in any 24 hour period, nor more than 24 hours in any 48 hour period, nor more than 72 hours in any 7 day period, all excluding shift turnover time;
3. A break of at least 8 hours should be allowed between work periods, including shift turnover time;
4. The use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized in advance by the Plant General Manager or, in his absence, Shift Manager, or by higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Routine deviation from the above guidelines is not authorized.

- f. The Shift Manager shall be a CERTIFIED FUEL HANDLER.
 - g. The Shift Managers shall report to an individual who is a CERTIFIED FUEL HANDLER.
-

5.0 ADMINISTRATIVE CONTROLS

5.4 Training

- 5.4.1 An NRC-approved retraining and replacement training program for the CERTIFIED FUEL HANDLERS shall be maintained under the direction of the Plant General Manager.
-

5.5 Review and Audits

5.5.1.1.3 Responsibilities (continued)

verify that such actions do not involve a change to the Technical Specifications or constitute an unreviewed safety question as defined in 10 CFR 50.59.

- b. Proposed changes to the programs required by specification 5.7.2, to verify such changes do not involve a change to the Technical Specifications and will not constitute an unreviewed safety question as defined in 10 CFR 50.59.
- c. Proposed changes to the Technical Specification Bases.

5.5.2 Independent Review and Audit Committee

The Independent Review and Audit Committee (IRAC) is responsible for reviewing and advising the Plant General Manager on matters relating to safe storage of irradiated fuel. This review and audit function is independent of the line organization responsibilities.

5.5.2.1 Composition

- 5.5.2.1.1 The IRAC shall be composed of a minimum of 5 members. Alternates may be substituted for regular members. The Plant General Manager shall designate in writing the chairman, the members, and alternates for the IRAC. The chairman shall not have management responsibilities for, or report to, the line organizations responsible for operation or maintenance of the facility.

- 5.5.2.1.2 The IRAC shall collectively have experience and knowledge in the following functional areas:

- 1. Fuel Handling & Storage
- 2. Chemistry and Radiochemistry
- 3. Engineering
- 4. Radiation Protection
- 5. Quality Assurance

(continued)

5.5 Review and Audits (continued)

5.5.2.1.3 Meeting Frequency

The IRAC shall hold meetings as required, with at least one meeting per quarter.

5.5.2.1.4 Quorum

A quorum shall consist of 3 regular members or duly appointed alternates. Those members representing the line organizations responsible for the operation and maintenance of the facility shall not constitute a majority of the quorum. At least one member of the quorum shall be the chairman or the chairman's designated alternate.

5.5.2.1.5 Functions

The IRAC shall, as a minimum, incorporate the following functions that:

- a. Advise the Plant General Manager on all matters related to safe storage of irradiated fuel;
- b. Advise the management of the audited organization and the Plant General Manager, of audit results as they relate to safe storage of irradiated fuel;
- c. Recommend to the management of the audited organization, and its management, any corrective action to improve the safe storage of irradiated fuel; and
- d. Notify the Senior Vice President, Power Supply of any safety significant disagreement between the IRAC and the Plant General Manager within 24 hours.

5.5.2.2 Review Responsibilities

The IRAC shall be responsible for the review of:

- a. The safety evaluations for procedures, and changes thereto, completed under the provisions of 10 CFR 50.59, to verify that such actions do not constitute an unreviewed safety question as defined in 10 CFR 50.59. This review may be completed after implementation of the affected procedure;

(continued)

5.5 Reviews and Audits

5.5.2.2 Review Responsibilities (continued)

- b. Changes to structures, systems, or components important to the safe storage of irradiated fuel to verify that such changes do not constitute an unreviewed safety question as defined in 10 CFR 50.59. This review may be completed after implementation of the change;
- c. Tests or experiments involving the safe storage of irradiated fuel to verify that such tests or experiments do not constitute an unreviewed safety question as defined in 10 CFR 50.59. This review may be completed after performance of the test or experiment;
- d. Proposed changes to these Technical Specifications or the License;
- e. Violations of codes, regulations, orders, license requirements, or internal procedures/instructions having nuclear safety significance;
- f. Indications of unanticipated deficiencies in any aspect of design or operation of structures, systems, or components that could affect safe storage of irradiated fuel;
- g. Significant accidental, unplanned, or uncontrolled radioactive releases, including corrective action to prevent recurrence;
- h. Significant operating abnormalities or deviations from normal and expected performance of equipment that affect safe storage of irradiated fuel;
- i. The performance of the corrective action system; and
- j. Internal and external experience information related to the safe storage of irradiated fuel that may indicate areas for improving facility safety.

Reports or records of these reviews shall be forwarded to the Plant General Manager within 30 days following completion of the review.

(continued)

5.5 Reviews and Audits (continued)

5.5.2.3 Audit Responsibilities

The audit responsibilities shall encompass:

- a. The conformance of irradiated fuel storage to provisions contained within the TS and applicable license conditions;
- b. The training and qualifications of the facility staff;
- c. The implementation of all programs required by Specification 5.7.2;
- d. Actions taken to correct deficiencies occurring in structures, systems, components, or method of operation that affect safe storage of irradiated fuel;
- e. Facility operations, modifications, maintenance, and surveillance related to the safe storage of irradiated fuel to verify independently that these activities are performed safely and correctly; and
- f. Other activities and documents as requested by the Plant General Manager.

Reports or records of these audits, including any recommendations for improving the safe storage of irradiated fuel, shall be forwarded to the Plant General Manager within 30 days following completion of the audit.

5.5.3 Records

Written records of reviews and audits shall be maintained. As a minimum these records shall include:

- a. Results of the activities conducted under the provisions of Section 5.5.1 and 5.5.2;
- b. Recommendations to the management of the organization being audited;

(continued)

5.0 ADMINISTRATIVE CONTROLS

5.7 Procedures, Programs, and Manuals

5.7.1 Procedures

5.7.1.1 Scope

Written procedures shall be established, implemented, and maintained covering the following activities:

- a. The procedures applicable to the safe storage of irradiated fuel recommended in Regulatory Guide 1.33, Revision 2, Appendix A, February 1978;
- b. Defueled security plan implementation;
- c. Defueled emergency plan implementation;
- d. Quality assurance for radiological effluent and environmental monitoring;
- e. Fire protection program implementation; and
- f. All programs specified in Specification 5.7.2.

5.7.1.2 Review and Approval

Each procedure of Specification 5.7.1.1, and changes thereto, shall be independently reviewed in accordance with established administrative procedures and approved by the Plant General Manager or his designee prior to implementation.

5.7.1.3 Temporary Changes

Temporary changes to procedures of Specification 5.7.1.1 may be made provided:

- a. The intent of the existing procedure is not altered;
- b. The change is approved by a member of the facility management staff and by a CERTIFIED FUEL HANDLER; and

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.2 Process Control Program (PCP) (continued)

- b. Shall be effective after review and approval by an Independent Safety Reviewer and the approval of the Plant General Manager or designee.

5.7.2.3 Offsite Dose Calculation Manual (ODCM)

5.7.2.3.1 Content

- a. The ODCM shall contain the methodology and parameters used in the calculation of off-site doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm and trip setpoints, and in the conduct of the Radiological Environmental Monitoring Program; and
- b. The ODCM shall also contain the Radioactive Effluent Controls Program and the Radiological Environmental Monitoring Program required by Specifications 5.7.2.4 and 5.7.2.5 respectively, and descriptions of the information that should be included in the Annual Radiological Environmental Monitoring Report required by Specification 5.8.1.2.

5.7.2.3.2 Licensee initiated changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained. This documentation shall contain:
 - 1. sufficient information to support the change(s) together with the appropriate analyses or evaluations justifying the change(s); and
 - 2. a determination that the change(s) maintain the levels of radioactive effluent control required by 10 CFR 20.1302, and 40 CFR 190, and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations;

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.3.2 Licensee initiated changes to the ODCM: (continued)

- b. Shall become effective after review and approval by an Independent Safety Reviewer and the approval of the Plant General Manager or designee; and
- c. Shall be submitted to the NRC in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radiological Environmental Monitoring Report for the period of the report in which any change in the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

5.7.2.4 Radioactive Effluent Controls Program

This program provides controls for radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as reasonably achievable. The program shall be contained in the ODCM, shall be implemented by procedures, and shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- a. Limitations on the functional capability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM;
- b. Limitations on the concentrations of radioactive material released in liquid effluents to unrestricted areas, conforming to 10 CFR 20, Appendix B;
- c. Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the ODCM;
- d. Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least quarterly;

(continued)