

LICENSEE EVENT REPORT (LER)

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|-----------------------------------------------------------------|--------------------------------------|----------------------|
| FACILITY NAME (1) Pilgrim Nuclear Power Station - Unit No. 1 | DOCKET NUMBER (2) 0 5 0 0 0 2 9 3 | PAGE (3) 1 OF 0 2 |
|-----------------------------------------------------------------|--------------------------------------|----------------------|

TITLE (4)
Failure to Meet Technical Specifications Requirements

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|------------------|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | DOCKET NUMBER(S) |
| 0 7 | 1 5 | 8 5 | 8 5 | 0 1 8 | 0 0 0 | 8 1 | 4 8 | 5 | | 0 5 0 0 0 |

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|-------------------------|--------------------------------------------------------------------------------------------------------------|--|------------------|------------------|--|----------------------|----------------------|--|--|--------------------------------------------------------------|--|--|
| OPERATING MODE (9) N | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) | | | | | | | | | | | |
| | 20.402(b) | | | 20.405(e) | | | 50.73(a)(2)(iv) | | | 73.71(b) | | |
| | 20.405(a)(1)(i) | | | 50.36(c)(1) | | | X 50.73(a)(2)(v) | | | 73.71(c) | | |
| | 20.405(a)(1)(ii) | | | 50.36(c)(2) | | | 50.73(a)(2)(vii) | | | OTHER (Specify in Abstract below and in Text, NRC Form 366A) | | |
| | 20.405(a)(1)(iii) | | | X 50.73(a)(2)(i) | | | 50.73(a)(2)(viii)(A) | | | | | |
| 20.405(a)(1)(iv) | | | 50.73(a)(2)(ii) | | | 50.73(a)(2)(viii)(B) | | | | | | |
| 20.405(a)(1)(v) | | | 50.73(a)(2)(iii) | | | 50.73(a)(2)(ix) | | | | | | |

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|----------------------------------------------|--|--|--|--|--|--|--|--------------------|--|
| LICENSEE CONTACT FOR THIS LER (12) | | | | | | | | | |
| NAME Gregory G. Belmonte - Plant Engineer | | | | | | | | TELEPHONE NUMBER | |
| | | | | | | | | AREA CODE 6 1 7 | |
| | | | | | | | | 7 4 6 - 7 9 0 0 | |

| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | | |
|----------------------------------------------------------------------------|--------|-----------|-------------------|------------------------|--|-------|--------|-----------|-------------------|------------------------|
| CAUSE | SYSTEM | COMPONENT | MANUFAC- Turer | REPORTABLE TO NPRDS | | CAUSE | SYSTEM | COMPONENT | MANUFAC- Turer | REPORTABLE TO NPRDS |
| B | B D | D M P | P 1 Q 1 4 | N | | | | | | |

| | | | | | | | | | |
|-------------------------------------------------|--|--|--|--|--|-------------------------------------|-------|-----|------|
| SUPPLEMENTAL REPORT EXPECTED (14) | | | | | | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
| YES (If yes, complete EXPECTED SUBMISSION DATE) | | | | | | | | | |
| X NO | | | | | | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 7/15/85, at approximately 1500 hrs., a secondary containment damper (AON-90) would not fully close. Subsequently, Technical Specification (T.S.) compensatory actions were not taken for approximately 19 hrs. Reactor power was approximately 100% prior to the event.

The cause of not performing the T.S. compensatory actions for 19 hrs. was due to utility Maintenance personnel error.

Corrective action was to align the drive gears on damper AON-90 and counsel personnel involved.

A search of our records indicates no previous LER's of a similar nature have been submitted.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

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|----------------------------------------------------------------------|----------------------------------------------|----------------|----------------------|--------------------|----------------------------|--|--|
| FACILITY NAME (1) Pilgrim Nuclear Power Station Unit No. 1 | DOCKET NUMBER (2) 0 5 0 0 0 2 9 3 8 5 | LER NUMBER (6) | | | PAGE (3) 0 2 OF 0 2 | | |
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| | | 8 5 | 0 1 8 | 0 0 | | | |

TEXT (If more space is required, use additional NRC Form 368A's) (17)

On 7/15/85, at approximately 1500 hrs., during a post-secondary containment isolation (Ref.: LER 85-017) inspection of the secondary containment dampers (EIIS Code DB), Damper AON-90 in the refuel floor exhaust duct would not fully close due to misalignment of the drive gears (EIIS Code DMP). Subsequently, compensatory actions required by Technical Specification (T.S.) 3.7.C.2 and 1.0.N. were not taken until approximately 19 hrs. after discovery of the damper problem. Reactor power was approximately 100% power prior to this event.

Cause of not performing the T.S. compensatory actions for approximately 19 hrs. was due to utility Maintenance personnel error. The error occurred when personnel failed to report the event according to Procedure 1.3.24 (Failure and Malfunction Report) which would have alerted the Watch Engineer that appropriate T.S. action was needed. The cause of the damper not fully closing was due to slippage in the drive gears.

Corrective action was to counsel involved personnel in the requirements of Procedure 1.3.24 (Failure and Malfunction Report).

A search of our records indicates no previous LER's of a similar nature have been submitted.

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

August 14, 1985
BECO Ltr. #85-149

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Washington, D.C. 20555

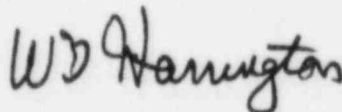
Docket Number 50-293
License DPR-35

Dear Sir:

The attached Licensee Event Report 85-018-00, "Failure to Meet Technical Specification Requirements," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,



W. D. Harrington

GB:caw

Enclosure: LER 85-018-00

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Standard BECO LER Distribution

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