

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Joseph M. Farley - Unit 1 DOCKET NUMBER (2) 0 5 0 0 0 3 4 8 PAGE (3) 1 OF 0 1

TITLE (4) Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 7	1 7	8 5	8 5	0 1 3	0 0 0	8 1	2 8	5			0 5 0 0 0
										0 5 0 0 0	

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																																			
1	<table border="1"><tr><td>20.402(b)</td><td></td><td>20.405(c)</td><td><input checked="" type="checkbox"/></td><td>50.73(a)(2)(v)</td><td></td><td>73.71(b)</td></tr><tr><td>20.405(a)(1)(i)</td><td></td><td>50.36(c)(1)</td><td></td><td>50.73(a)(2)(vi)</td><td></td><td>73.71(c)</td></tr><tr><td>20.405(a)(1)(ii)</td><td></td><td>50.36(c)(2)</td><td></td><td>50.73(a)(2)(vii)</td><td></td><td>OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td></tr><tr><td>20.405(a)(1)(iii)</td><td></td><td>50.73(a)(2)(i)</td><td></td><td>50.73(a)(2)(viii)(A)</td><td></td><td></td></tr><tr><td>20.405(a)(1)(iv)</td><td></td><td>50.73(a)(2)(ii)</td><td></td><td>50.73(a)(2)(viii)(B)</td><td></td><td></td></tr><tr><td>20.405(a)(1)(v)</td><td></td><td>50.73(a)(2)(iii)</td><td></td><td>50.73(a)(2)(ix)</td><td></td><td></td></tr></table>										20.402(b)		20.405(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(v)		73.71(b)	20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(vi)		73.71(c)	20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)	20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)		
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LICENSEE CONTACT FOR THIS LER (12)  
NAME J. D. Woodard TELEPHONE NUMBER  
AREA CODE 2 0 5 8 9 9 - 5 1 5 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)  
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO  
EXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 2218 on 7-17-85, during steady state operation at 100 percent power, a reactor trip occurred due to low-low water level in the 1B steam generator. The low-low steam generator level occurred because the 1B steam generator feed pump (SGFP) tripped due to a spurious indication of excessive thrust bearing wear. A technician working in a main control board cabinet accidentally bumped a cable and broke a connection on a wire leading to the 1B SGFP thrust bearing wear protection system. The broken connection caused the bearing wear protection system to indicate excessive thrust bearing wear and the 1B SGFP tripped. Following the trip of the SGFP, the operator attempted to prevent the reactor trip by reducing reactor power, but he was unable to reduce power below the capacity of a single SGFP before the low-low steam generator level was reached.

Following the reactor trip, the operators implemented FNP-1-EFP-0 (Reactor Trip or Safety Injection) and FNP-1-ESP-0.1 (Reactor Trip Response), ensuring that the unit was safely in Mode 3. All safety systems functioned as designed.

The cable was replaced and the technician involved in this event was counseled. Power operation was resumed on 7-18-85. Health/safety of the public was not affected by this event.

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PDR ADOCK 05000348  
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**Mailing Address**

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600 North 18th Street  
Post Office Box 2641  
Birmingham, Alabama 35291  
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**R. P. McDonald**  
Senior Vice President  
Flintridge Building



Alabama Power  
the southern electric system

August 12, 1985

Docket No. 348

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Sir:

Joseph M. Farley Nuclear Plant, Unit 1, Licensee Event Report  
No. LER 85-013-00 is forwarded in accordance with 10CFR50.73 to provide 30  
day written notification of the occurrence.

If you have any questions, please advise.

Yours very truly,

R. P. McDonald

RPM/DSM:sam

Enclosure

xc: IE, Region II

IE22  
1/1