

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Browns Ferry - Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 2 9 6 1 OF 0 2										PAGE (3) 1 OF 0 2	
TITLE (4) Failed Supports on the Residual Heat Removal System																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)								
0 7	1 6	8 5	8 5	0 1 7	0 0 0	8 0	9 8	5					0 5 0 0 0								
OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)																			
POWER LEVEL (10) 0 1 0 0		20.402(b)				20.406(c)				50.73(a)(2)(iv)				73.71(b)							
		20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)							
		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
		20.406(a)(1)(iii)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)(A)											
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)											
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																					
NAME Patrick N. Ebersole										TELEPHONE NUMBER AREA CODE 2 0 5 7 2 9 1 - 3 7 8 8											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPD											
X	EK	BKR	G O 8 1 0	Y																	
X	BO	H	T 1 4 2	Y																	
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
X YES (If yes, complete EXPECTED SUBMISSION DATE)										NO		1	1	5 8 5							

ABSTRACT (Limit to 1400 spaces - approximately fifteen single-space typewritten lines) (16)

On July 16, 1985, while in cold shutdown, loop I of the unit 3 residual heat removal (RHR) system was declared inoperable following discovery of a broken hanger and a damaged snubber on the torus return line. Technical Specifications require that two RHR pumps and their associated diesel generator be operable when irradiated fuel is in the reactor, and the vessel pressure is atmospheric. One of the diesel generators associated with the loop II RHR pumps had been previously declared inoperable due to a failed generator field breaker. Maintenance activities were immediately initiated to return the inoperable diesel to service. The supports for the 18-inch test return line were repaired and returned to service on July 27, 1985. An engineering evaluation is continuing to determine the cause of the damaged hangers.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Browns Ferry - Unit 3	0500029685	01	7	00	02	OF 02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Units 1 and 2 were in a refueling outage, and unit 3 was in an extended maintenance outage. The condition described below is applicable only to unit 3 which was in cold shutdown.

On July 16, 1985, at 1415, loop I of the residual heat removal (RHR) system was declared inoperable after a broken hanger and a damaged snubber on the torus return line were discovered by personnel working in the vicinity. Prior to discovery of the broken hanger, loop I of the RHR system had been in shutdown cooling. The hanger, designated H3, and the snubber, designated R41, provide vertical support for the 18-inch torus return line. These devices have been inspected in June of this year as part of the torus attached piping supports reinspection program. At the time of this reinspection, no problems with these devices were reported. A small broken hanger was also found on July 12, 1985, on the one-inch vent line that comes off HCV 74-55. HCV 74-55 is the inboard manual isolation valve for loop I of the RHR system piping.

Technical Specifications require that two RHR pumps and their associated diesel generator be operable when irradiated fuel is in the reactor, and the vessel pressure is atmospheric. The emergency diesel generator for one of the two RHR pumps on loop II had been previously declared inoperable pending delivery and installation of a replacement field breaker. Maintenance activities were immediately initiated to return the inoperable diesel generator to service utilizing a field breaker removed from another diesel generator. The diesel generator was successfully tested and declared operable at 2140 on July 16, 1985. This returned the number of operable RHR pumps and associated diesel generators to that required by technical specifications for unit 3.

The small hanger that attached the one-inch vent line to HCV 74-55 was modified and returned to service on August 2, 1985. The support for the 18-inch test return line was repaired and returned to service on July 27, 1985. The failure mechanism for these support is still under evaluation. The results of this evaluation will be provided in an update of this LER.

Two RHR pumps and one emergency diesel were available throughout this situation. The inoperable diesel was restored to service the same day. Emergency diesel power was also available from unit 1, and the RHR loop which had been declared inoperable was available for emergency duty. The hangers were not damaged while the reactor was in operation. Therefore, no actual safety problems were created.

Responsible Plant Section - N/A

Previous Events - None

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant

P. O. Box 2000

Decatur, Alabama 35602

August 9, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 3 -
DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE
OCCURRENCE REPORT BFRO-50-296/85017

The enclosed report provides details concerning failure supports on the
residual heat removal system. This report is submitted in accordance
with 10 CFR 50.73(a)(2)(i).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

Robert L. Lewis
Robert L. Lewis
Acting Plant Manager
Browns Ferry Nuclear Plant

Enclosures

cc (Enclosures):

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NRC Resident Inspector, BFN

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