

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Shoreham Nuclear Power Station Unit #1

DOCKET NUMBER (2)

0 5 0 0 0 3 2 2 1 OF 0 2

PAGE (3)

TITLE (4)

Failure of Primary Containment Personnel Hatch Full Volume Leak Test

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																
0	7	1	3	8	5	8	5	0	2	6	0	0	0	8	0	9	8	5			0	5	0	0	0		

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10)	0 1 0 1 0	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)			
		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)			
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		X OTHER (Specify in Abstract below and in Text, NRC Form 366A)			
		20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		Voluntary			
		20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)					
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)  
NAME  
Gary G. Rhoads, Operational Compliance Engineer

TELEPHONE NUMBER

AREA CODE

5 1 1 6 9 1 2 1 9 1 8 1 3 1 0 1 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)  
☐ YES (If yes, complete EXPECTED SUBMISSION DATE) ☒ NO

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 13, 1985 at 1710 P.M. while performing a Surveillance Leak Rate Test of the Primary Containment Personnel Hatch (full volume test), the leakage rate was found to be in excess of the allowable limit of the Technical Specification (3/4.6.1.1). The plant was in Operational Condition 3, 0 PSIG, 210 degrees F, with the Reactor Mode Switch in Shutdown, cooling down to reach Operational Condition 4. Ten minutes after finding the leakage the plant was in Operational Condition 4.

An investigation determined the leakage to be from the Secondary Side, Door Interlock Pressure Switch and from the through door penetrations for 1T23\*01V-3012 & 3013 (equalizing valves). The Pressure Switch was repaired, the valves were seated, and the hatch was successfully retested.

Although the Action Statement of the applicable Technical Specification was met the Licensee determined a voluntary report was warranted.

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S PDR

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104  
EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Shoreham Nuclear Power Station Unit #1	0500032285	—	026	—	00	2	OF 02

TEXT (If more space is required, use additional NRC Form 306a's) (17)

On July 13, 1985 at 1710 P.M. while performing a Surveillance Leak Rate Test of the Primary Containment Personnel Hatch (full volume test), the leakage rate was found to be in excess of the allowable limit of the Technical Specification (3/4.6.1.1). The plant was in Operational Condition 3, 0 PSIG, 210 Degrees F, with the Reactor Mode Switch in Shutdown. At 1720 P.M. the plant was in Operational Condition 4.

The Technical Limit for leakage is 337 Standard Cubic Feet per Day (SCFD) and the actual leakage measured was 1272.96SCFD. An investigation into the source of the leakage indicated that the leakage was from the Secondary Side, Door Interlock Pressure Switch and from the through - door penetrations for 1T23\*01V-3012 & 1T23\*01V-3013 (equalizing valves). The manual valves, which are cycled when personnel use the Personnel Access Hatch were seated after they were found to be not tightly seated. The Secondary Side, Door Interlock Pressure Switch was repaired per Maintenance Work Request, MWR 85-4066. A retest was performed verifying leakage to be within the limits of the Technical Specification.

Plant Management was notified of the event. There was no safety significance to the event. All plant systems functioned as designed. No ECCS systems were challenged or required for the event. The operators carried out all required actions.

Although the Action Statement of the applicable Technical Specification was met the Licensee determined a voluntary report was warranted.

To prevent recurrence of this event the following corrective action was taken:

- 1) Instruction Notices were installed in the vicinity of the Personnel Air Locks exhibiting the proper operation and valving of the Air Lock.
- 2) A plant Safety Directive was issued requiring personnel knowledgeable in Personnel Air Lock operation to be in the air lock when people are using the hatch to enter the Drywell.



## LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION • P.O. BOX 628 • WADING RIVER, NEW YORK 11792

TEL (516) 929-8300

August 9, 1985

PM 85-147

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Dear Sir:

In accordance with 10CFR50.73, enclosed is a copy of Shoreham Nuclear Power Station Unit 1's License Event Report 85-026.

Sincerely yours,

William E. Steiger, Jr.  
Plant Manager

WES/gr

Enclosure

cc: Dr. Thomas E. Murley, Regional Administrator  
John Berry, Senior Resident Inspector  
Institute of Nuclear Power Operations, Records Center  
American Nuclear Insurers

SR. A21.0200

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