

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Wolf Creek Generating Station										DOCKET NUMBER (2) 0 5 0 0 0 4 8 1 2				PAGE (3) 1 OF 0 2								
TITLE (4) Engineered Safety Features Actuations - Control Room Ventilation Isolations																						
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)									
0	7	1	2	8	5	8	5	0	5	2	0	0	0	8	0	9	8	5	0 5 0 0 0			
OPERATING MODE (9) 1			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																			
POWER LEVEL (10) 0 1 7 1 5			20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)							
			20.405(a)(1)(i)				50.36(a)(1)				<input type="checkbox"/> 50.73(a)(2)(v)				73.71(c)							
			20.405(a)(1)(ii)				50.36(a)(2)				<input type="checkbox"/> 50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
			20.405(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(viii)(A)											
			20.405(a)(1)(iv)				50.73(a)(2)(ii)				<input type="checkbox"/> 50.73(a)(2)(viii)(B)											
			20.405(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(x)											
LICENSEE CONTACT FOR THIS LER (12)																						
NAME Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services										TELEPHONE NUMBER												
										AREA CODE 3 1 1 6 3 1 6 4 1 - 1 8 1 8 1 3 1 1												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS												
B	V	I	A	I	T	I	X	1	9	1	9	N										
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR						
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO										

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 12, 1985, at approximately 0445 CDT, and on July 22, 1985, at approximately 0353 CDT, Control Room Ventilation Isolation Signals were initiated due to an alarm on a Control Room Heating, Ventilation, and Air Conditioning chlorine monitor. During the first event, the plant was in Mode 3, Hot Standby, and during the second event, the plant was in Mode 1, Power Operation, at a reactor power level of 75 percent.

In both cases, the alarm was due to a broken filter paper tape, which was replaced. Investigations after each event revealed no specific cause for the tape breakage, but the paper paths and flow block were cleaned subsequent to the July 22 event as a precautionary measure.

All Engineered Safety Features responded properly on both occasions, and no abnormal chlorine levels were present as evidenced by the redundant chlorine monitor.

There was no damage to plant equipment or release of radioactivity as a result of this event, and at no time was there a threat to the health and safety of the public.

8508200159 850809  
PDR ADOCK 05000482  
S PDRIE22  
111

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)  Wolf Creek Generating Station	DOCKET NUMBER (2)  0 5 0 0 0 4 8 2 8 5 - 0 5 2 - 0 0 0 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	OF	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On July 12, 1985, at approximately 0445 CDT, and on July 22, 1985, at approximately 0353 CDT, Control Room Ventilation Isolation Signals (CRVIS) were initiated due to an alarm on GK-AITS3, a Control Room Heating, Ventilation, and Air Conditioning chlorine monitor [VI-AIT].

At the time of the July 12 occurrence, the plant was in Mode 3, Hot Standby, with the Reactor Coolant System [AB] at normal operating temperature and pressure. At 0445 CDT, the CRVIS was initiated by a High Chlorine alarm on GK-AITS3, and all Engineered Safety Features equipment responded properly.

A broken filter paper tape was identified as being the cause of the alarm. The actuated systems were returned to normal configuration at 0601 CDT. The filter paper tape was replaced, and GK-AITS3 was returned to service at 0606 CDT.

No definite cause for the filter tape breakage could be determined during subsequent investigations. At that time, it was felt that the most probable cause was a faulty tape.

On July 22, 1985, with the plant in Mode 1, Power Operation, at a reactor power level of approximately seventy-five percent, another CRVIS occurred due to a High Chlorine alarm on GK-AITS3 at 0353 CDT, and all Engineered Safety Features equipment responded properly.

It was determined that a broken filter paper tape was the cause of the alarm. The tape was replaced, and GK-AITS3 was returned to service at 0424 CDT. Restoration from the CRVIS was completed at 0508 CDT.

Investigations following the July 22 event revealed no specific cause for tape breakage. The filter paper paths and flow block that the paper rides on were cleaned as a precautionary measure, although no dirt or misalignment was detected in the monitor.

The chlorine monitor was supplied by MDA Scientific Inc. (Model 7040 FAN) and uses a cassette type filter paper tape cartridge. Although faulty paper tapes could have contributed to the occurrence of these events, no definite cause has been identified.

In both instances, there were no chlorine levels above normal as evidenced by the redundant chlorine monitor, GK-AITS2. There was no damage to plant equipment or release of radioactivity as a result of these events, and at no time did conditions develop that may have posed a threat to the public health or safety.



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER  
VICE PRESIDENT - NUCLEAR

August 9, 1985

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Mr. R.P. Denise, Director  
Wolf Creek Task Force  
U.S. Nuclear Regulatory Commission  
Region IV  
611 Ryan Plaza Drive, Suite 1000  
Arlington, Texas 76011

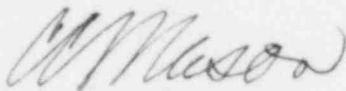
KMLNRC 85-194  
Re: Docket No. STN 50-482  
Subj: Licensee Event Report 85-052-00

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a) (2) (iv) concerning an Engineered Safety Feature actuation.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

*for*   
Glenn L. Koester  
Vice President - Nuclear

GLK:dab

Enclosure

xc: PO'Connor (2), w/a  
JCummins, w/a

IE22  
1/1