

U. S. NUCLEAR REGULATORY COMMISSION

REGION III

Reports No. 50-440/85034; 50-441/85016

Docket Nos. 50-440; 50-441

Licenses No. CPPR-148; CPPR-149

Licensee: Cleveland Electric Illuminating Company
Post Office Box 5000
Cleveland, Ohio 44101

Facility Name: Perry Nuclear Power Plants, Units 1 and 2

Inspection At: Perry Site, Perry, Ohio

Inspection Conducted: May 30 through July 26, 1985

Inspector: D. E. Keating

Approved By: *RC Knop*
R. C. Knop, Chief
Reactor Projects Section 1C

8-15-85
Date

Inspection Summary

Inspection on May 30 through July 26, 1985 (Reports No. 50-440/85034(DRP);
No. 50-441/85016(DRP))

Areas Inspected: Review of documentation concerning 10 CFR 50.55(e), inspection and documentation review of instrumentation cables and terminations, witnessing of drywell structural integrity test, review of procedures for program covering extended construction delay, and plant tours. The inspection covered 136 hours onsite by the resident inspector including 35 off-shift hours.

Results: No violations were identified. No significant safety issues were identified during this inspection period.

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DETAILS

Prepared By: D. E. Keating
Reviewed By: R. C. Knop

1. Persons Contacted

Cleveland Electric Illuminating Company

*C. M. Shuster, Manager, Nuclear Quality Assurance Department

*E. Riley, Supervisor, Quality Assurance, Construction
Quality Section

*K. Cimorelli, Lead Quality Engineer, Electrical Unit,
Construction Quality Section

E. Parker, Unit Supervisor, Piping, Instrumentation, and
Controls, Construction Quality Section

E. C. Christiansen, Senior Engineer, Nuclear Construction
Engineering Section

Weiss, Janney, and Elstner

G. Hedien, Senior Engineer

D. Heidbrink, Engineer

Gilbert Associates, Inc.

C. Angstadt, Senior Structural Engineer

The inspector also contacted and interviewed other licensee and contractor personnel during this inspection period.

*Denotes those attending one or more of the exit meetings.

2. Licensee Action on 10 CFR 50.55(e) Items (92703)

(Closed) 10 CFR 50.55(e) - (440/84043-EE)(DAR 207): Potential Deficiency of Fuel Control Levers on Transamerica Delaval Standby Diesel Generators. The inspector reviewed the letter from Transamerica Delaval notifying the NRC and the licensee of a 10 CFR Part 21 concerning the identification of certain Standby Diesel engines supplied to a number of construction sites without 1/4" diameter retaining pins. The lack of these pins plus the loss of proper cap screw tension could lead to engine unavailability. The inspector also reviewed the licensee's notification to the Region of the potential 10 CFR 50.55(e) concerning this item.

The licensee conducted a thorough inspection of this concern and determined that the cap screws associated with the control shaft levers and injection pump levers were torqued to the manufacturer's recommendations and that all retaining pins were in place as required. In order to assure the availability of these engines through surveillance of this condition, the licensee wrote a nonconformance (OQS-1173) and dispositioned it to track the adjustment and torquing of cap screws on the control shaft levers and injection pump levers, and to assure that all pins are in place. Based on the inspection referred to above and these additional actions, the licensee had withdrawn the potential 10 CFR 50.55(e).

This item is, therefore, considered closed.

3. Instrument Cable and Terminations (52063B)

The inspector physically inspected pulled stainless steel jacketed cables for circuits 1-D19-R1A, 4A, 11B, and 14B for Divisions I and II. Also reviewed was the documentation concerning this activity. As indicated, cabling for four (4) circuits had been pulled and terminated. Cabling for eight (8) more circuits needs to be installed. These cables have not yet arrived onsite. The inspector will be notified when they do arrive in order to witness receipt inspection activities prior to witnessing pulling and termination.

The documentation for the four (4) pulled circuits appeared to be in order. Nonconformances (NRs), Field Variance Authorizations (FVAs), and Engineering Change Notices (ECNs) clearly stated the conditions to be addressed and adequately dispositioned. The necessary signatures were present and the dates of signatures were in proper sequence. These activities will continue to be monitored during future inspections.

4. Status of Nuclear Grade Coatings (92706)

During a walkdown of the Drywell in conjunction with the Structural Integrity Test, the inspector observed two (2) SRV downcomers that had exposed areas of metal. Upon closer observation there were two (2) areas, one (1) above, the other below, the lugs of a two (2) unit snubber support and several areas along the fillet welds of the snubber support lugs where the coatings had either rubbed off or buckled due to pipe movement.

This condition was brought to the licensee's attention. The licensee was already aware of this and several other areas where this condition exists. NRs have been written and dispositioned to change the coating. Test samples of the new coating have been made and site exposure tests are nearly complete. The samples of the new coating have been made and site exposure tests are nearly complete. The samples will then be sent to Oakridge, Tennessee for nuclear evaluation and qualification. This item will continue to be monitored during future inspections. This is considered an open item (440/85034-01(DRP)).

5. Drywell Structural Integrity Test (63700)

A regional based inspector and the resident inspector accompanied licensee personnel on several walkdowns as they attempted to perform the Drywell Structural Integrity Test. Problems were encountered with a boot-type flexible connection on the SRV piping and their associated penetrations through the Drywell wall into the suppression pool. The problems concerned the rupturing of the boot material during pressurization. NRs were written covering a total of 13 of these boots. After three (3) previous attempts resulting in boot failures, the test was suspended during the fourth attempt at 4:01 a.m. on July 27, 1985 after attaining 18.7 psig. The test will be performed at a later date in conjunction with or during a window in the Loss of Offsite Power Verification Test.

The inspectors also reviewed the accumulated structural data. This will be addressed in more detail in Regional Inspection Report 50-440/85048.

6. Review of Quality Assurance For Extended Construction Delay (92050)

The inspector has reviewed Procedure PNPP 5541 and PPD Interface Document 7-0301. These documents present the maintenance program that controls the activities pertaining to periodic inspection of Unit 2 equipment. A walkdown of equipment and areas is planned in the near future in order to establish a reference point for the prescribed periodic inspections required in I.P. 92050.

7. Plant Tours and Housekeeping (92706)

The inspector made numerous tours of the Control Complex, Containment, and Drywell, noting the conditions in these areas after temporary demobilization for the pending series of tests and the nature of work remaining to be completed. Overall housekeeping has improved greatly and is being maintained on a continuous basis. It is expected that this level of activity will continue.

8. Open Items

Open items are matters which have been discussed with the licensee, which will be reviewed further by the inspector, and which involve some action on the part of the NRC or licensee or both. An Open Item is identified in Paragraph 4 of this report.

9. Exit Interviews (30703)

The inspector met singularly and in conjunction with a region based inspector, the persons noted in Paragraph 1 on July 2 and August 1, 1985 to discuss the scope and results of the inspections. The licensee did not identify any proprietary concerns relating to the inspection.